

SEMIANALYTICAL SOLUTIONS OF RADIOACTIVE OR REACTIVE TRANSPORT IN VARIABLY-FRACTURED LAYERED MEDIA: 1. SOLUTES

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Abstract. In this paper, semianalytical solutions are developed for the problem of transport of radioactive or reactive solute tracers through a layered system of heterogeneous fractured media with misaligned fractures. The tracer transport equations in the non-flowing matrix account for (a) diffusion, (b) surface diffusion, (c) mass transfer between the mobile and immobile water fractions, (d) linear kinetic or equilibrium physical, chemical, or combined solute sorption or colloid filtration, and (e) radioactive decay or first-order chemical reactions. The tracer-transport equations in the fractures account for the same processes, in addition to advection and hydrodynamic dispersion. Any number of radioactive decay daughter products (or products of a linear, first-order reaction chain) can be tracked. The solutions, which are analytical in the Laplace space, are numerically inverted to provide the solution in time and can accommodate any number of fractured and/or porous layers. The solutions are verified using analytical solutions for limiting cases of solute and colloid transport through fractured and porous media. The effect of important parameters on the transport of ^3H , ^{237}Np and ^{239}Pu (and its daughters) is investigated in several test problems involving layered geological systems of varying complexity.

1. Introduction

The study of radioactive and/or reactive contaminant transport in complex fractured geologic systems has become increasingly important in recent years because of the need to predict the migration and fate of the contaminants. Currently, there are some very

large contaminated sites (such as Hanford, Washington; Nevada Test Site (NTS), Nevada; Idaho National Engineering and Environmental Laboratory (INEEL), Idaho) where severe pollution by radioactive materials extends over large areas within the subsurface rocks.

At Yucca Mountain (YM), Nevada, the site of the potential repository for high-level nuclear waste, the transport of radioactive contaminants must be predicted for tens to hundreds of thousands of years. Performing reliable radionuclide transport calculations for this temporal and spatial scale is obviously very difficult, and furthermore it is impossible to verify the results. In addition, the complex geology of the site and the unsaturated nature of a significant portion of the flow path add to the difficulty in making such predictions.

The potential site is located in southern Nevada about 120 km northwest of Las Vegas, and is characterized by a thick unsaturated zone (600–700 m) and the presence of rocks onto which important radionuclides in the wastes tend to sorb strongly. The YM stratigraphy consists of layers of welded and nonwelded tuffs (with vastly different hydraulic, transport, and geochemical properties), with the former generally being extensively fractured and the latter behaving similarly to a porous medium [*Montazer and Wilson, 1984; Liu et al., 1998; Bandurraga and Bodvarsson, 1999*].

The varied geological and hydrological characteristics of the different tuff layers at Yucca Mountain make the modeling of flow and transport a challenging task. A single representation for all of the hydrogeologic units is inappropriate, and several different approaches and algorithms must be employed for obtaining reliable modeling results. Analytical and semianalytical models of transport that can account for the site heterogeneity are important because they allow the validation of complex multidimensional numerical models, are computationally efficient, and can provide bounding estimates of the possible solutions of the expected transport at the site.

Previous analytical solutions of solute transport in fractured media involved exclusively single semi-infinite domains (layers). *Tang et al.* [1981] developed a quasi two-dimensional solution for the transport of solutes in a single saturated fracture (i.e., with a semi-infinite

matrix) that assumed a constant concentration boundary and accounted for (a) advection and dispersion in the fractures, (b) diffusion in the matrix, the fractures, and across their interface, (c) sorption onto the matrix and the fractures, and (d) radioactive decay. The analytical solution of *Sudicky and Frind* [1982] accounted for the same processes in a system of parallel fractures (i.e., with a finite matrix block size). The solution of *Robinson et al.* [1998] is an extension of the *Sudicky and Frind* [1982] solution and accounts for the effect of fracture skin on transport in a system of parallel fractures. By neglecting hydrodynamic dispersion in the fractures and assuming an instantaneous (Dirac-type) deposition of a parent radionuclide at the boundary, *Sudicky and Frind* [1984] obtained analytical solutions to the problem of transport of a two-member radioactive chain in a single fracture.

In this paper, semianalytical solutions are developed for the problem of transport of radioactive or reactive solute tracers (i.e., at concentrations that do not affect the fluid properties) through a layered system of heterogeneous fractured media with misaligned fractures (such as the unsaturated zone at YM). The solutions allow any number and combination of fractured and/or porous layers that can vary in hydraulic and transport properties, fracture frequency, water saturation, fracture flow, and fracture-matrix interaction. The tracer transport equations in the non-flowing matrix account for (a) molecular diffusion, (b) surface diffusion, (c) mass transfer between the mobile and immobile water fractions, (d) linear kinetic or equilibrium physical, chemical or combined solute sorption, and (e) radioactive decay or first-order chemical reactions. The solute transport equations in the fractures account for the same processes, in addition to advection and hydrodynamic dispersion. Any number of daughter products of radioactive decay (or of a linear, first-order reaction chain) can be tracked, and several boundary conditions can be accommodated.

2. Solute Transport Equations

2.1. The PDE of Solute Transport

The one-dimensional (1-D) Partial Differential Equation (PDE) of transport of a

radioactive or reactive solute tracer through a variably saturated porous or fractured medium (PM or FM) is described by the equation

$$\begin{aligned}
 & D_m \frac{\partial^2 C}{\partial x^2} + D_i \frac{\partial^2 C_i}{\partial x^2} + D_F \frac{\partial^2 F}{\partial x^2} - U \frac{\partial C}{\partial x} \\
 & = \phi (S - S_r) \left(\frac{\partial C}{\partial t} + \delta_r \frac{\partial \mathcal{R}}{\partial t} \right) + \phi S_r \left(\frac{\partial C_i}{\partial t} + \delta_r \frac{\partial \mathcal{R}_i}{\partial t} \right) + (1 - \phi) \rho \frac{\partial F}{\partial t} \quad (1) \\
 & \quad + \lambda \delta_\lambda [\phi (S - S_r) C + \phi S_r C_i + (1 - \phi) \rho F],
 \end{aligned}$$

where

- C dissolved species concentration in the mobile pore water [ML^{-3}];
- D_m intrinsic diffusion coefficient for the mobile pore water [L^2T^{-1}];
- C_i dissolved species concentration in the immobile pore water [ML^{-3}];
- D_i intrinsic diffusion coefficient in the immobile pore water [L^2T^{-1}];
- $F = F_p + F_c$;
- F_p relative concentration of the physically adsorbed species [$(ML^{-3})/(ML^{-3})$];
- F_c relative concentration of the chemically sorbed species [$(ML^{-3})/(ML^{-3})$];
- \mathcal{R} reacted species mass per unit volume in the mobile fraction [ML^{-3}];
- \mathcal{R}_i reacted species mass per unit volume in the immobile fraction [ML^{-3}];
- D_F apparent surface diffusion coefficient [$ML^{-1}T^{-1}$];
- $U = V \phi (S - S_r)$, Darcy velocity [LT^{-1}];
- V pore flow velocity [LT^{-1}];
- S water saturation [L^3/L^3];
- S_r irreducible water saturation [L^3/L^3];
- ρ PM grain density [ML^{-3}];
- ϕ total PM porosity [L^3/L^3];
- $\lambda = \ln 2/T_{1/2}$, radioactive decay constant [T^{-1}];
- $T_{1/2}$ half-life of radioactive species [T].

The parameters δ_s and δ_λ are defined as

$$\delta_r = \begin{cases} 1 & \text{for reactive transport} \\ 0 & \text{for radionuclide transport} \end{cases} \quad \text{and} \quad \delta_\lambda = \begin{cases} 0 & \text{for reactive transport} \\ 1 & \text{for radionuclide transport} \end{cases}$$

The first three terms on the left-hand side of (1) describe diffusion in the mobile pore water [Skagius and Neretnieks, 1988] through the immobile thin film in the immediate vicinity of the PM grains [de Marsily, 1986], and surface diffusion [Jahnke and Radke, 1987; Skagius and Neretnieks, 1988; Cook, 1989; Berry and Bond, 1992], respectively. The fourth term on the left-hand side (1) describes advective transport. The terms on the right-hand side of equation (1) describe the dissolved species accumulation and radioactive decay in the pore water, in the immobile fraction, and on the PM grains due to sorption. Chemical reactions in the water phase are also accounted for [Cho, 1971]. A detailed discussion of these terms can be found in Moridis [1999], from where

$$D_m = \phi (S - S_r) (\tau_p D_0 + \alpha_L V) \quad \text{and} \quad D_i = \tau_i \phi S_r D_0 \quad (2)$$

where D_0 is the molecular diffusion coefficient of the dissolved species in water [L^2T^{-1}], α_L is the longitudinal dispersivity [L], τ_p is the tortuosity factor of the pore paths [*dimensionless*], and τ_i is the tortuosity factor in the diffusion paths through the immobile fraction [*dimensionless*]. If surface diffusion cannot be neglected [Jensen and Radke, 1988], D_F is given by [Jahnke, 1986; Jahnke and Radke, 1987]

$$D_F = \tau_s (1 - \phi) \rho D_s, \quad (3)$$

where τ_s is the tortuosity coefficient of the surface path [*dimensionless*], and D_s is the surface diffusion coefficient [L^2T^{-1}]. For homogeneous PM systems there is theoretical justification [Cook, 1989] for the relationship $\tau_s = \frac{2}{3} \tau_p$.

The species concentration in the mobile and immobile water fractions are related through the linear equilibrium relationship [de Marsily, 1986],

$$C_i = K_i C, \quad \mathcal{R}_i = K_i \mathcal{R}, \quad (4)$$

where K_i is a dimensionless mass transfer coefficient. Equation (1) then becomes

$$\begin{aligned} D_T \frac{\partial^2 C}{\partial x^2} + D_F \frac{\partial^2 F}{\partial x^2} - U \frac{\partial C}{\partial x} \\ = \phi h \left(\frac{\partial C}{\partial t} + \delta_\lambda \lambda C \right) + (1 - \phi) \rho \left(\frac{\partial F}{\partial t} + \delta_\lambda \lambda F \right) + \delta_r \phi h \frac{\partial \mathcal{R}}{\partial t}, \end{aligned} \quad (5)$$

where

$$D_T = \phi \{D_0 [\tau_p (S - S_r) + \tau_i S_r K_i] + (S - S_r) \alpha_L V\} \quad (6)$$

and

$$h = (S - S_r) + S_r K_i. \quad (7)$$

2.2. The Equations of Solute Sorption and First-Order Chemical Reaction

Considering that sorption occurs as the dissolved species diffuse through the immobile water fraction, and assuming linear equilibrium (LE) sorption, the following relationship applies:

$$F_p = K_d K_i C, \quad (8)$$

where K_d is the distribution coefficient [$M^{-1}L^3$].

Linear kinetic physical (LKP) and linear irreversible physical (LIP) sorption are described by the equation [Moridis, 1999]

$$\frac{\partial F_p}{\partial t} + \lambda F_p = k_p (K_d K_i C - \delta_p F_p), \quad (9)$$

where k_p is the kinetic constant of linear adsorption [T^{-1}], and

$$\delta_p = \begin{cases} 1 & \text{for LKP sorption;} \\ 0 & \text{for linear LIP sorption.} \end{cases} \quad (10)$$

In the case of LIP sorption, K_d does not represent the distribution coefficient of LE sorption, but is rather a proportionality factor.

The first-order reversible chemical sorption is represented by the linear kinetic chemical (LKC) model

$$\frac{\partial F_c}{\partial t} + \lambda F_c = k_c^+ K_i C - k_c^- F_c, \quad (11)$$

where k_c^+ [$M^{-1}L^3T^{-1}$] and k_c^- [T^{-1}] are the forward and backward kinetic constants, respectively. Note that equation (11) can be used in conjunction with the physical sorption

equations to describe combined sorption [*Cameron and Klute, 1977*], e.g., physical and chemical sorption. Combined sorption accounts for the different rates at which a species is sorbed onto different PM constituents. Thus, sorption onto organic components may be instantaneous (LE), while sorption onto mineral surfaces may be much slower and kinetically controlled [*Cameron and Klute, 1977*].

The equations of a series of N_c first-order chemical reaction are given by [*Cho, 1971*]

$$\begin{aligned} \frac{\partial \mathcal{R}_1}{\partial t} &= \mathcal{K}_1 C_1, \\ \frac{\partial \mathcal{R}_2}{\partial t} &= \mathcal{K}_2 C_2 - \mathcal{K}_1 C_1, \\ &\vdots \\ \frac{\partial \mathcal{R}_{N_c}}{\partial t} &= \mathcal{K}_{N_c} C_{N_c} - \mathcal{K}_{N_c-1} C_{N_c-1}, \end{aligned} \quad (12)$$

where \mathcal{K}_j ($j = 1, \dots, N_c$) is the chemical reaction rate constant [T^{-1}], and N_c is the number of chemical reactions in the series.

2.3. The Solute Transport ODE in the Laplace Space

2.3.1. Parent or Stable Species. After incorporating the sorption terms, the Laplace transform (LT) of the solute transport equation (5) yields the following Ordinary Differential Equation (ODE)

$$D \frac{d^2 \widehat{C}}{dx^2} - U \frac{d\widehat{C}}{dx} - E \widehat{C} = 0, \quad (13)$$

where $\widehat{C} = \mathcal{L}\{C\}$, $\mathcal{L}\{\}$ denotes the LT of the quantity in the brackets,

$$\begin{aligned} E &= \phi [(s + \delta_\lambda \lambda) R + \delta_r h \mathcal{K}], \quad (14) \\ R &= \begin{cases} h + w \psi & \text{for LE sorption;} \\ h + u \psi & \text{for LKP or LIP sorption,} \\ h + v \psi & \text{for LKC sorption,} \\ h + (w + u) \psi & \text{for combined LE and LKP/LIP sorption,} \\ h + (w + v) \psi & \text{for combined LE and LKC sorption,} \\ h + (u + v) \psi & \text{for combined LKP/LIP and LKC sorption,} \end{cases} \quad (15) \end{aligned}$$

$$D = \begin{cases} D_T + \phi \tau_s w \psi D_s & \text{for LE sorption;} \\ D_T + \phi \tau_s u \psi D_s & \text{for LKP or LIP sorption,} \\ D_T + \phi \tau_s v \psi D_s & \text{for LKC sorption,} \\ D_T + \phi \tau_s (w + u) \psi D_s & \text{for combined LE and LKP/LIP sorption,} \\ D_T + \phi \tau_s (w + v) \psi D_s & \text{for combined LE and LKC sorption,} \\ D_T + \phi \tau_s (u + v) \psi D_s & \text{for combined LKP/LIP and LKC sorption,} \end{cases} \quad (16)$$

$$w = K_d K_i, \quad u = \frac{k_p K_d K_i}{s + \lambda + \delta_p k_p}, \quad v = \frac{k_c^+ K_i}{s + \lambda + k_c^-}, \quad \psi = \frac{(1 - \phi)}{\phi} \rho, \quad (17)$$

and s is the Laplace space parameter. The term R is an expanded retardation factor, which can account for kinetic behavior [Moridis, 1999]. Its development involves the LT of the sorption from equations (8) through (11). It is straightforward to show that [Moridis, 1998]

$$\widehat{F} = p \widehat{C} \quad (18)$$

where $\widehat{F} = \mathcal{L}\{F\}$ and

$$p = \begin{cases} w & \text{for LE sorption;} \\ u & \text{for LKP or LIP sorption,} \\ v & \text{for LKC sorption,} \\ w + u & \text{for combined LE and LKP/LIP sorption,} \\ w + v & \text{for combined LE and LKC sorption,} \\ u + v & \text{for combined LKP/LIP and LKC sorption.} \end{cases} \quad (19)$$

Equation (13), subject to equations (14) through (19), is the Laplace space equation of solute transport in its most general form. Implicit in (13) are the assumptions that (a) $C(x, t = 0) = 0$, (b) $F(x, t = 0) = 0$, (c) $\mathcal{R}(x, t = 0) = 0$, and (d) in combined sorption, different sites are involved in each of the constituent types of sorption.

2.3.2. Daughter Species of Radioactive Decay. If the species is radioactive, the right-hand side of equation (5) is augmented by the term

$$-\lambda_{\nu-1} m_r [\phi h C_{\nu-1} + (1 - \phi) \rho F_{\nu-1}], \quad \text{where} \quad m_r = \frac{\mathcal{M}_{\nu}}{\mathcal{M}_{\nu-1}},$$

\mathcal{M}_ν is the molecular weight of the ν -th daughter ($1 < \nu \leq N_d$, N_d being the total number of radioactive decay or reaction products), and $\nu - 1$ refers to the decaying parent. Then, the Laplace space transport equation for any daughter product ν of the decay chain following a LE isotherm is given by

$$D_\nu \frac{d^2 \widehat{C}_\nu}{dx^2} - U \frac{d \widehat{C}_\nu}{dx} - E_\nu \widehat{C}_\nu = -G_\nu \widehat{C}_{\nu-1}, \quad (20)$$

where

$$G_\nu = \phi m_r \lambda_{\nu-1} R_{\nu-1} \quad (21)$$

If the daughter sorption is kinetically controlled, equations (9) and (11) need to account for the generation of daughter mass due to the decay of the sorbed parent, and become

$$\frac{\partial F_\nu}{\partial t} + \lambda_\nu F_\nu - \lambda_{\nu-1} m_r \zeta_\nu F_{\nu-1} = k_\alpha C_\nu - k_\beta F_\nu, \quad (22)$$

where $F_{\nu-1}$ is the sorbed mass of the parent,

$$k_\alpha = \begin{cases} k_p K_d K_i & \text{for LKP/LIP sorption,} \\ k_c^+ K_i & \text{for LKC sorption,} \end{cases} \quad k_\beta = \begin{cases} k_p \delta_p & \text{for LKP/LIP sorption,} \\ k_c^- & \text{for LKC sorption,} \end{cases}$$

and ζ_ν is the fraction of the mass of the decayed sorbed parent that remains sorbed as a daughter ($0 \leq \zeta_\nu \leq 1$). The term ζ_ν is introduced to account for the possibility that daughters can be ejected from grain surfaces due to recoil, e.g., the ejection of ^{234}Th from grain surfaces during the alpha decay of ^{238}U [Faure, 1977]. The LT of (22) returns

$$\widehat{F}_\nu = p \widehat{C}_\nu + m_r p_r \widehat{C}_{\nu-1}, \quad (23)$$

where p is obtained from equation (19), and

$$p_r = \begin{cases} \frac{\lambda_{\nu-1} \zeta_\nu u}{s + \lambda_\nu + k_p \delta_p} & \text{for (a) LKP/LIP or (b) combined LE-LKP/LIP sorption} \\ \frac{\lambda_{\nu-1} \zeta_\nu v}{s + \lambda_\nu + k_c^-} & \text{for (a) LKC sorption or (b) combined LE-LKC sorption} \end{cases} \quad (24)$$

For combined LKC and LKP/LIP sorption, p_r is the sum of the two components in (24).

Using (23) and (24), it is easy to show that equation (20) applies, but with

$$G_\nu = \phi m_r [\lambda_{\nu-1} R_{\nu-1} - (s + \lambda_\nu) p_r]. \quad (25)$$

All other terms in (20) remain unchanged. Equations (20) through (25) are valid in any layer n . For a complete daughter ejection [Faure, 1977], $\zeta_\nu = 0$, $p_r = 0$, and (21) and (25) become identical.

2.3.3. Products of Chemical Reactions. If the species is a product of the ν -th first-order chemical reaction in the reaction chain (12), the right-hand side of equation (5) is augmented by the term $-\phi h \mathcal{K}_{\nu-1} C_{\nu-1}$. Then, equation (20) applies unchanged, but with

$$G_{\nu-1} = \phi h \mathcal{K}_{\nu-1}. \quad (26)$$

3. Transport in Layered Fractured Media

The development of the equations for transport in a layered fractured media expands on the analysis of Tang *et al.* [1981] and Sudicky and Frind [1982]. A schematic of the fracture-matrix system is shown in Figure 1, in which the N layers have different properties.

3.1. Transport in the Matrix

3.1.1. The ODE of Parent or Stable Species Transport in the Matrix. Advection in the matrix is neglected, that is $U_n^m = 0$. Then the Laplace space ODE of the species transport in the matrix layer n is given by

$$D_n^m \frac{d^2 \widehat{C}_n^m}{dx_n^2} - E_n^m \widehat{C}_n^m = 0, \quad (27)$$

where the superscript m denotes the matrix. The diffusive flux across the fracture-matrix interface is given by

$$q_n = -r_n D_n^m \left. \frac{\partial C_n^m}{\partial x_n} \right|_{x_n = 0} \quad (28)$$

and differs from the analogous expression of Tang *et al.* [1981] in the inclusion of the active interface area reduction factor r_n . The term r_n ($1 \geq r_n > 0$) is defined as the ratio of the average interface area between mobile water in a fracture and its surrounding matrix to the average interface area between a fracture and the surrounding matrix. A detailed discussion on the subject can be found in Liu *et al.* [1998]. For a fully saturated fracture, $r_n = 1$.

3.1.2. The ODE of Daughter Transport in the Matrix. From equation (20), the Laplace space ODE of transport of the daughter ν in the matrix of layer n is given by

$$D_{n,\nu}^m \frac{d^2 \widehat{C}_{n,\nu}^m}{dx_n^2} - E_{n,\nu}^m \widehat{C}_{n,\nu}^m = -G_{\nu}^m \widehat{C}_{n,\nu-1}^m, \quad (29)$$

where the term G_{ν}^m is computed from (21) to (26). The diffusive flux of the daughter ν across the fracture-matrix interface is given by equation (28).

3.2. Transport in the Fractures

3.2.1. Adjustments to Concepts and Equations. In fracture transport, the Darcy velocity U_n in any layer n is computed from the basic mass balance equation as

$$U_n = \frac{Q_w}{M_n b_n},$$

where Q_w is the water influx rate per unit fracture thickness (in the y direction, not shown in Figure 1) at the $z_1 = 0$ boundary [$L^2 T^{-1}$], and $2b_n$ is the fracture aperture [L]. The parameter M_n [L/L] is the relative fracture density, and is determined from the number of fractures in an arbitrary length L_x (see Figure 1). The term L_x is related to the matrix block half-width X_n [L] and b_n (see Figures 2a and 2b) through the relationship

$$M_n = \frac{L_x}{2(X_n + b_n)}, \quad n = 1, \dots, N.$$

There are two different ways to treat the fractures. If the fractures are open, we have surface-based rather than volume-based sorption in the fractures of any layer n ($n = 1, \dots, N$). The following changes are then made:

- (a) F is now the mass of solute adsorbed per unit surface of the fracture [ML^{-2}].
- (b) From the mass balance equations, the term $(1 - \phi) \rho$ in (17) is replaced by $1/b_n$, where b_n is the fracture half-width or half-aperture [L] in layer n .
- (c) The distribution coefficient of the fracture K_d^f is now defined as the mass of solute adsorbed per unit area of surface divided by the concentration of solute in solution [Tang *et al.*, 1981], with units [L].

- (d) The kinetic constants k_c^+ of chemical sorption in (11) have units $[LT^{-1}]$; k_c^- in (11) have units $[ML^{-2}T^{-1}]$.

If the fractures are filled (a rather common occurrence), they are treated as a porous medium. Then, there is no need for the conceptual or mathematical adjustments in (1) through(4). In both open and filled fractures, the right-hand side of equation (5) is augmented by the term

$$Q_n = f_n^q q_n, \quad \text{where} \quad f_n^q = \begin{cases} 1/b_n & \text{for open fractures} \\ 1 & \text{for filled fractures,} \end{cases} \quad (30)$$

and q_n is described by (28).

3.2.2. The ODE of Parent or Stable Species Transport in the Fractures. The Laplace space equation for fracture transport along the z -coordinate (Figure 1) then becomes

$$D_n^f \frac{d^2 \widehat{C}_n^f}{dz_n^2} - U_n \frac{d\widehat{C}_n^f}{dz_n} - E_n^f \widehat{C}_n^f = \widehat{Q}_n, \quad (31)$$

where the f superscript denotes the fracture, the n subscripts denotes the layer, and $\widehat{Q}_n = \mathcal{L}\{Q_n\}$. Equation (31) is written in terms of the local coordinate z_n in each layer n .

3.2.3. The ODE of Daughter Transport in the Fractures. The Laplace space ODE of transport for the daughter ν in the matrix of layer n is given by

$$D_{n,\nu}^f \frac{d^2 \widehat{C}_{n,\nu}^f}{dz_n^2} - U_n \frac{d\widehat{C}_{n,\nu}^f}{dz_n} - E_{n,\nu}^f \widehat{C}_{n,\nu}^f = \widehat{Q}_n - G_n^f \widehat{C}_{n,\nu-1}^f. \quad (32)$$

All the terms in (32) are as previously defined.

3.3. Initial and Boundary Conditions

The initial and boundary conditions corresponding to the fracture equation are

$$\begin{aligned} C_n^f(z_n, t = 0) &= 0, \\ C_1^f(z_1 = 0, t) &= C_{z_0}(t), \\ C_n^f(z_n = Z_n, t) &= C_{n+1}^f(z_{n+1} = 0, t), \quad n = 1, \dots, N-1, \\ C_N^f(z_N \rightarrow \infty, t) &= 0, \end{aligned} \quad (33)$$

where Z_n denotes the thickness of the n -th segment (layer). The time dependence of C_{z_0} allows investigation of systems with time-variable upper boundaries. Some of the more common forms of $C_{z_0}(t)$ are

$$C_{z_0}(t) = \begin{cases} C_0 & \text{constant concentration} \\ C_0 \exp[-\lambda(t + t_d)] & \text{decaying radionuclide concentration} \\ \sum_{i=1}^{N^*} C_i^* [\mathcal{U}(t - t_{i-1}^*) - \mathcal{U}(t - t_i^*)] & \text{variable pulse concentration} \end{cases} \quad (34)$$

where C_0 is a constant, t_d is the release delay (the time between radionuclide generation or storage, and the beginning of release), $\mathcal{U}(t - t^*)$ denotes the unit step function at time t^* , and N^* is the number of the different pulses with concentration C_i^* . Note that $t_0^* = 0$ and that, for $N^* = 1$, we obtain the unit pulse of duration t_1^* .

The initial and boundary conditions corresponding to the matrix equation are

$$\begin{aligned} C_n^m(x, t = 0) &= 0, \\ C_n^m(x = 0, t) &= C_n^f(z_n, t), \\ \begin{cases} \frac{\partial C_n^m}{\partial x}(x = X, t) = 0 & \text{for Case 1 (Figure 2a),} \\ C_n^m(x \rightarrow \infty, t) = 0 & \text{for Case 2 (Figure 2b),} \end{cases} \end{aligned} \quad (35)$$

where X is the half-width of the matrix block (Figure 2). Case 1 in Figure 2a describes a finite system with a Neuman-type boundary at $x = X$. If dry fractures (i.e., fractures in which the water phase is discontinuous) occur in the rock matrix of Case 1, the half-width X is replaced by $X^* = 2X/(n_d + 1)$, where n_d is the number of dry fractures evenly spaced along x in the matrix block (Figure 2b). Case 2 in Figure 2b describes a semi-infinite system. The Laplace transforms of equations (33) through (35) are trivial.

4. The Laplace Space Equations

4.1. General Matrix Solutions in Each Layer

4.1.1. Parent or Stable Species. Omitting for simplicity the n subscript, and expanding on *Tang et al.* [1981] and *Sudicky and Frind* [1982], the solutions to (27) are

$$\widehat{C}^m = \begin{cases} H^c \cosh[\theta(X - x)] & \text{for Case 1} \\ H^e \exp(-\theta x) & \text{for Case 2} \end{cases} \quad (36)$$

respectively, where H^c and H^e are parameters to be determined, and

$$\theta = \theta(s) = \sqrt{\frac{E^m}{D^m}}. \quad (37)$$

From (36) and the Laplace transform of (35),

$$\widehat{C}^m(x=0) = \begin{cases} H^c \cosh(\theta X) = \widehat{C}^f \Rightarrow H^c = \frac{\widehat{C}^f}{\cosh(\theta X)} & \text{for Case 1} \\ H^e = \widehat{C}^f & \text{for Case 2} \end{cases} \quad (38)$$

from which

$$\widehat{C}^m = \widehat{C}^m(x, s) = \begin{cases} \frac{\cosh[\theta(X - x)]}{\cosh(\theta X)} \widehat{C}^f & \text{for Case 1} \\ \exp(-\theta x) \widehat{C}^f & \text{for Case 2} \end{cases} \quad (39)$$

The equations in (39) are applicable in any layer n ($n = 1, \dots, N$).

4.1.2. Daughter or Reaction Products. Following the same approach, it is straightforward to show that the Laplace space solution of the ODE in (32) for any daughter or reaction product ν is given by

$$\widehat{C}_\nu^m = \begin{cases} H_\nu^c \cosh[\theta_\nu(X - x)] + \sum_{\kappa=\nu-1}^1 \left(\prod_{i=\nu}^{\kappa} A_{i\kappa}^m \right) H_\kappa^c \cosh[\theta_\kappa(X - x)] & \text{for Case 1} \\ H_\nu^e \exp(-\theta_\nu x) + \sum_{\kappa=\nu-1}^1 \left(\prod_{i=\nu}^{\kappa} A_{i\kappa}^m \right) H_\kappa^e \exp(-\theta_\kappa x) & \text{for Case 2} \end{cases} \quad (40)$$

where

$$A_{i\kappa}^m = -\frac{G_i^m}{D_i^m \theta_\kappa^2 - E_i^m}. \quad (41)$$

The coefficients H_ν are given by the general expression

$$H_\nu = \sum_{\kappa=1}^{\nu} T_{\nu,\kappa} \widehat{C}_\kappa^f, \quad (42)$$

where $T_{\nu,\kappa}$ are appropriate coefficients. Expressions for H_ν and $T_{\nu,\kappa}$ (the derivation of which is tedious but straightforward) are provided in Appendix A. Equation (40) shows that the solution of the matrix transport equation of the daughter or reaction product ν requires knowledge of the fracture solutions of all previous members of the decay or reaction chain.

4.2. General Fracture Solutions in Each Layer

4.2.1. Parent or Stable Species. From the Laplace transform of the diffusive flux in (30), and omitting for simplicity the subscript n ,

$$\widehat{Q} = \gamma f^q \widehat{C}^f, \quad (43)$$

where

$$\gamma = \begin{cases} r D^m \theta \tanh(\theta X) & \text{for Case 1} \\ r D^m \theta & \text{for Case 2} \end{cases} \quad (44)$$

Substituting in (31) and collecting terms,

$$D^f \frac{d^2 \widehat{C}^f}{dz^2} - U \frac{d\widehat{C}^f}{dz} - E^* \widehat{C}^f = 0, \quad (45)$$

where $E^* = E^f + \gamma f^q$. The general solution to (45) is given by

$$\widehat{C}^f = \widehat{C}^f(x, s) = \alpha \exp(\eta^+ z) + \beta \exp(\eta^- z), \quad (46)$$

where α and β are parameters to be determined, and

$$\eta^\pm = \frac{U \pm \sqrt{U^2 + 4D^f E^*}}{2D^f}. \quad (47)$$

Equations (43)–(47) apply in any layer n .

4.2.2. Daughter or Reaction Products. From equations (30)–(32) and (40)–(42), for a daughter ν

$$\widehat{Q}_\nu = f^q r D_\nu^m W_\nu = f^q \sum_{\kappa=1}^{\nu} \gamma_{\nu,\kappa} \widehat{C}_\kappa^f. \quad (48)$$

Equation (48) is general and applies to both Case 1 and Case 2. Expressions for W_ν and $\gamma_{\nu,\kappa}$ are provided in Appendix B.

Substituting in (32) and collecting terms,

$$D_\nu^f \frac{d^2 \widehat{C}_\nu^f}{dz^2} - U \frac{d \widehat{C}_\nu^f}{dz} - E_\nu^* \widehat{C}_\nu^f = -G_\nu \widehat{C}_{\nu-1}^f + f^q \sum_{\kappa=1}^{\nu-1} \gamma_{\nu,\kappa} \widehat{C}_\kappa^f, \quad (49)$$

where $E_\nu^* = E_\nu^f + \gamma_{\nu,\nu} f^q$.

Following the same approach, it is straightforward to show that the Laplace space solution of any daughter or reaction product ν is given by

$$\widehat{C}_\nu^f = \alpha_\nu \exp(\eta_\nu^+ z) + \beta_\nu \exp(\eta_\nu^- z) + Y_\nu, \quad (50)$$

where

$$Y_\nu = \sum_{\kappa=\nu-1}^1 A_{\nu,\kappa}^+ \alpha_\kappa \exp(\eta_\kappa^+ z) + \sum_{\kappa=\nu-1}^1 A_{\nu,\kappa}^- \beta_\kappa \exp(\eta_\kappa^- z), \quad (51)$$

and

$$A_{\nu,\kappa}^\pm = \frac{B_{\nu,\kappa}^\pm}{D_\nu^f (\eta_\kappa^\pm)^2 - U \eta_\kappa^\pm - E_\nu^*}. \quad (52)$$

Expressions for $B_{\nu,\kappa}^\pm$ and for $\nu \leq 5$ are given in Appendix C. Equations (50) and (51) show that the solution of the fracture transport equation of the daughter or reaction product ν requires knowledge of all previous α_ν and β_ν , i.e., the solutions of all previous members of the chain.

5. The Solution Approach

5.1. Determination of the α and β Parameters

Equation (46) defines a total of $2N$ unknowns, i.e., the α and β parameters in each of the N subdomains. These are obtained from the solution of the following equations.

5.1.1. Boundary Equations. These apply to the $z_1 = 0$ point in the first layer ($n = 1$).

From (46) and the Laplace transform of (33), for a known boundary concentration we have

$$\alpha_1 + \beta_1 = \widehat{C}_{z_0}, \quad (53)$$

while for known flux boundary conditions

$$\alpha_1(U_1 - D_1^f \eta^+) + \beta_1(U_1 - D_1^f \eta^-) = U_1 \widehat{C}_{z_0} \quad (54)$$

where $\widehat{C}_{z_0} = \mathcal{L}\{C_{z_0}\}$. For the common boundary conditions in (34),

$$\widehat{C}_{z_0} = \begin{cases} \frac{C_0}{s} & \text{constant concentration} \\ \frac{C_0 \exp(-\lambda t_d)}{s + \lambda} & \text{decaying radionuclide concentration} \\ \sum_{i=1}^{N^*} \frac{C_i^*}{s} [\exp(-st_{i-1}^*) - \exp(-st_i^*)] & \text{piecewise constant concentration.} \end{cases} \quad (55)$$

For the limiting case of a system consisting of a single semi-infinite layer (i.e., $N = 1$) with an open fracture and a constant concentration at $z_1 = 0$, $\alpha_1 = 0$, $\beta_1 = C_0/s$, and equation (46) is reduced to the Laplace space solutions obtained by *Tang et al.* [1981] (Case 2) and *Sudicky and Frind* [1982] (Case 1).

5.1.2. Concentration Equations. At the layer interfaces we have the equations

$$\alpha_{n-1} \exp(\eta_{n-1}^+ Z_{n-1}) + \beta_{n-1} \exp(\eta_{n-1}^- Z_{n-1}) - \alpha_n - \beta_n = 0, \quad (56)$$

for $n = 2, \dots, N$. An additional equation is provided by the requirement that \widehat{C}_n^f be finite for $Z_N \rightarrow \infty$, which dictates that $\alpha_N = 0$.

5.1.3. Flux Equations. The remaining $N - 1$ equations are provided by the equality of fluxes across the layer boundaries in the fractures, which dictates that

$$M_{n-1} b_{n-1} \left[U_{n-1} C_{n-1}^f - D_{n-1}^f \frac{dC_{n-1}^f}{dz_{n-1}} \right]_{Z_{n-1}} = M_n b_n \left[U_n C_n^f - D_n^f \frac{dC_n^f}{dz_n} \right]_0, \quad (57)$$

in which the quantity in the brackets is computed at the value of the local z coordinate indicated by the bracket subscript. From (46) and (57) we obtain

$$\begin{aligned} & \alpha_{n-1} \left[M_{n-1} b_{n-1} (U_{n-1} - D_{n-1}^f \eta_{n-1}^+) \right] \exp(\eta_{n-1}^+ Z_{n-1}) \\ & + \beta_{n-1} \left[M_{n-1} b_{n-1} (U_{n-1} - D_{n-1}^f \eta_{n-1}^-) \right] \exp(\eta_{n-1}^- Z_{n-1}) \\ & - \alpha_n \left[M_n b_n (U_n - D_n^f \eta_n^+) \right] - \beta_n \left[M_n b_n (U_n - D_n^f \eta_n^-) \right] = 0 \end{aligned} \quad (58)$$

where $n = 1, \dots, N - 1$.

5.1.4. Equations for Daughters. For a daughter product ν of radioactive decay or reaction, the following changes are made to equations (53) through (58):

- (a) In the right-hand side of equations (53) and (54), the term \widehat{C}_{z_0} is replaced by \widehat{C}_{ν,z_0} , where $\widehat{C}_{\nu,z_0} = \mathcal{L}\{C_{\nu,z_0}\}$, and C_{ν,z_0} is the concentration of daughter ν at $z_1 = 0$. For a constant C_{ν,z_0} , \widehat{C}_{ν,z_0} can be obtained from equation (55). For a $z_1 = 0$ boundary with a decaying radionuclide concentration, \widehat{C}_{ν,z_0} is computed from the Laplace transform of the mass balance equation $\frac{\partial C_{\nu,z_0}}{\partial t} = \lambda_\nu C_{\nu,z_0} - \lambda_{\nu-1} C_{\nu-1,z_0}$ as

$$\widehat{C}_{\nu,z_0} = \frac{C_{\nu,z_0}}{s + \lambda_\nu} \exp(-\lambda t_d) + m_r \frac{\lambda_{\nu-1}}{s + \lambda_\nu} \exp(-\lambda t_d) \widehat{C}_{\nu-1,z_0}. \quad (59)$$

For a reaction chain, equation (59) indicates a recursive reaction.

- (b) The zero on the right-hand side of the layer interface equation (56) is replaced by $Y_{\nu,n}(z_n = 0) - Y_{\nu,n-1}(z_{n-1} = Z_n)$ for $n = 2, \dots, N$.
- (c) Equation (57) applies unchanged. The zero on the right-hand side of equation (58) is replaced by the known quantity

$$M_n b_n \left[U_n Y_{\nu,n} - D_{\nu,n}^f \frac{dY_{\nu,n}}{dz_n} \right]_0 - M_{n-1} b_{n-1} \left[U_{n-1} Y_{\nu,n-1} - D_{\nu,n-1}^f \frac{dY_{\nu,n-1}}{dz_{n-1}} \right]_{Z_{n-1}}$$

5.2. The Laplace Space Solutions

The generality and complexity of these equations preclude the development of closed-form solutions for α_i, β_i ($i = 1, \dots, N$). Consequently, it is not possible to analytically invert equations (46) or (50), and to obtain a closed-form equation for concentration in time. The problem is alleviated by numerically inverting the Laplace space solutions. The algebraic equations discussed in Section 5.1 may be written in a general matrix form as:

$$\mathbf{M} \vec{\mathbf{X}} = \vec{\mathbf{B}}, \quad (60)$$

where \mathbf{M} is the coefficient matrix, $\vec{\mathbf{X}}$ is the vector of the unknowns, and $\vec{\mathbf{B}}$ is the composite vector of knowns. Solution of (71) returns the vector

$$\vec{\mathbf{X}} = \begin{pmatrix} \vec{\mathbf{X}}_1 \\ \vec{\mathbf{X}}_2 \\ \vdots \\ \vec{\mathbf{X}}_N \end{pmatrix}, \quad \text{where} \quad \vec{\mathbf{X}}_i = \begin{pmatrix} \alpha_i \\ \beta_i \end{pmatrix}, \quad i = 1, \dots, N. \quad (61)$$

The solution of the matrix equation (60) necessitates arithmetic values for the s parameter of the Laplace space. These are provided by the numerical inversion scheme of *DeHoog et al.* [1982] that uses complex values for s . The quantities \mathbf{M} , $\vec{\mathbf{X}}$ and $\vec{\mathbf{B}}$ assume the complex type of s . A detailed discussion of the application of this method and its performance can be found in *Sudicky* [1990] and *Moridis* [1998].

The α_i and β_i computed from the matrix equation (60) are then used to obtain all the \widehat{C}_n^f solutions ($i = 1, \dots, N$). The corresponding \widehat{C}_n^m solutions are obtained from \widehat{C}_n^f and equations (39) or (40)–(42). Note that the solutions for daughters or reaction products require knowledge of the solutions of all the previous members in the chain.

5.3. Numerical Inversions of the Laplace Space Solutions

The various time-variable concentrations can be determined by numerically inverting the Laplace space solutions, i.e.,

$$C_n^f(x, t) = \mathcal{L}^{-1}\{\widehat{C}_n^f(x, s)\}, \quad C_n^m(x, t) = \mathcal{L}^{-1}\{\widehat{C}_n^m(x, s)\}, \quad (62)$$

where $\mathcal{L}^{-1}\{\}$ denotes the inverse Laplace transform of the quantity in the brackets. Details on the inversion will not be discussed here; they can be found in *DeHoog et al.* [1982].

6. Treatment of Special Conditions

6.1. Misaligned Fractures

The analysis presented thus far assumes that the effect of fracture offset on transport is negligible. This may not be the case for large fracture spacing or at short observation times.

The process that accounts for fracture misalignment is described in Figure 3. The increased travel path of the transporting water caused by the offset fractures is indicated by the horizontal pathway at the confluence of the n and $n + 1$ layers in Figure 3a, and its effect is described by the addition of an “interlayer”, i.e., a pseudo-layer (Figure 3b) with the following characteristics:

- (a) A thickness $Z_I = \max\{X_n, X_{n+1}\}$ if $M_n > M_{n+1}$, or $Z_I = \min\{X_n, X_{n+1}\}$ if $M_n < M_{n+1}$.
- (b) A relative frequency $M_I = M_n$.
- (c) An open or filled fracture of half-width b_I , through which water flows between the n and $n + 1$ layers. The properties of the fracture in the interlayer are independent of those in the layers above and below.
- (d) A complex matrix, composed of the matrices of both the n and $n + 1$ layers. In Figure 3b, the matrices of the n and $n + 1$ layers are positioned on the left and right sides of the fracture, respectively. The two components of the matrix are assumed to be semi-infinite, as illustrated by their rotation by 90° (with respect to the original layer orientation) in Figure 3b. Then, the flux into the composite matrix of the interlayer is computed from equation (43), but with $\gamma \equiv \gamma_I$, where

$$\gamma_I = \frac{1}{2} (\gamma_n + \gamma_{n+1}), \quad (63)$$

and γ_n, γ_{n+1} are computed from equation (44).

Thus, consideration of misaligned fracture transforms a system of N layers to a system of $N + N_I$ layers, where N_I is the number of interlayers. The solution of the augmented system does not pose any particular challenges and proceeds in the manner discussed in Section 5. Note that this approximation involves the longest possible travel path and the largest possible amount of tracer diffusion. This is because diffusion into the matrix of the interlayer (see Figure 3b) is larger than that into the n and $n + 1$ layers (along the layer interface) owing to steeper gradients and their semi-infinite nature. Thus, the assumption of fracture alignment provides the most conservative solution, while the assumption of fracture

misalignment (as described by the concept of interlayers) reflects the least conservative scenario. These two solutions provide the limits that bracket the true solution.

6.2. Occasional Unfractured Layers

If the layered system includes unfractured (porous) layers (e.g., Layer 3 in Figure 1), these are treated as a combination of a pseudo-matrix (representing the nonflowing portion of the layer) and a pseudo-fracture representing the flowing portion of the layer. In essence, unfractured layers are treated as filled-fracture systems, and all the equations apply unchanged. The properties of the unfractured medium are assigned to both the pseudo-matrix and the pseudo-fracture. The relative sizes of b and X can describe the flowing and non-flowing portions of the porous medium. If water flows uniformly through the porous medium, $X = 0$. This approach maintains water mass and flux balance.

It is obvious that, for unfractured media, $L_x = 2(b_n + X_n)$, i.e., $M_n = 1$. Note that water saturations S must be obtained from the solution of the steady-state flow equation because the derivation of the transport equations is based on time-invariant flow conditions and cannot compute changes in S .

6.3. Transport in Layered Unfractured Media

This is a limiting case of the scenario discussed in Section 6.2. Setting the non-flowing portion of the matrix $X_n = 0$ ($n = 1, \dots, N$) transforms the problem into that of 1-D solute transport in a layered porous (unfractured) system. Then, all the solutions derived here apply unchanged.

7. Verification

A FORTRAN program was written to obtain the semianalytical (SA) solutions developed in Sections 4 through 6 by first solving (60), and then performing the numerical inversion indicated in (62). This code, named FRACL, accounts for all the processes,

phenomena and conditions discussed in Sections 2 through 6. It can obtain solutions for a system involving an arbitrary number of layers N of any combination of porous and/or fractured media, and up to 4 daughters. It is computationally very efficient, and required less than 10 seconds for any of the problems discussed in Sections 7 or 8.

FRACL is verified through comparisons to analytical solutions of radioactive solute and colloid transport in 1-D porous (unfractured) media and quasi-2-D fractured media. In all cases, FRACL solutions are first obtained in a system consisting of a single semi-infinite layer (i.e., $N = 1$). The domain is then subdivided into three layers in the z direction, and FRACL solutions for this multilayered system ($N = 3$) are obtained. Coincidence of the analytical solutions to the FRACL solutions for $N = 1$ and for $N = 3$ verifies FRACL.

7.1. Tests FS1 and FS2: Radioactive Solute Transport in Fractures

Tests FS1 and FS2 describe transport with LE sorption in the fracture-matrix system of Case 1 (parallel fractures, Figures 2a) and Case 2 (single fracture, Figure 2b), respectively. The corresponding analytical solutions were developed by *Sudicky and Frind* [1982] and *Tang et al.* [1981]. The values of the parameters used for the computation of the analytical and the SA solutions are in *Sudicky and Frind* [1982], and are listed in Table 1. A constant concentration (CC) condition is applied at $z_1 = 0$.

Figure 4 shows the distribution of the relative concentration C_R (defined as $C_R = C_n^f/C_{z0}$) in the fractures along the z axis at (a) $t = 1,000$ days in Test FS1 and (b) $t = 10,000$ days in Test FS2. In both tests, the analytical solution and the two FRACL solutions (for $N = 1$ and $N = 3$) are identical in the first 5 significant digits.

7.2. Tests PS1 to PS4: Radioactive Solute Transport in Unfractured Porous Media

Tests PS1 to PS4 are designed to confirm the ability of the SA solutions to describe transport in unfractured media without any modification. The solution to this problem is provided by *Bear* [1979], and accounts for LE sorption and radioactive decay.

The values of the parameters used for the computation of the analytical and the SA solutions of Tests PS1 to PS4 are listed in Table 2. In all four tests, a constant concentration condition is applied at $z_1 = 0$. The solute is a nondecaying isotope in Tests PS1 and PS2, and a decaying radionuclide in Tests PS3 and PS4. LE sorption is considered in Tests PS2 and PS3, but is ignored in Tests PS1 and PS4.

Figure 5 shows the distribution of the relative concentration C_R along the z axis at $t = 200$ days. The SA predictions of C_R distributions for both $N = 1$ and $N = 3$ are identical with the analytical solutions of *Bear* [1979].

7.3. Test PS5: Transport of a Three-Member Radioactive Solute Chain in Unfractured Porous Media

This test is designed to verify the ability of the SA solutions to describe the transport of reactive chains in unfractured media without any modification. An analytical solution to this problem was developed by *Harada et al.* [1980], and accounts for LE sorption, radioactive decay, and time-variable boundary conditions.

Test PS5 describes the transport of the radioactive chain $^{234}\text{U} \rightarrow ^{230}\text{Th} \rightarrow ^{226}\text{Ra}$ through a sorbing porous medium. The concentration of ^{234}U (i.e., the parent radionuclide) at the $z_1 = 0$ is not constant over time, but subject to radioactive decay. The initial concentrations of the ^{230}Th and ^{226}Ra daughter radionuclides at the $z_1 = 0$ boundary are zero, but increase over time because of the decay of their parents.

The values of the parameters used for the computation of the analytical and the SA solutions of Test PS5 are as in *Harada et al.* [1980], and are listed in Table 3. Figure 6 shows that the analytical solutions at $t = 10,000$ years coincide with the SA predictions (for both $N = 1$ and $N = 3$) of the C_R distributions of the three radioactive chain members.

8. Analysis and Test Problems

In this section the transport of various radionuclides is studied in layered systems

(involving both fractured and porous layers) of different characteristics and properties. The D_0 and λ of the radionuclides discussed here appear in Table 4.

8.1. Problem 1: Importance of Fracture Misalignment

This problem studies the importance of fracture misalignment on transport, as quantified by the concept of interlayers (discussed in Section 6.1). The following analysis focuses on the effects of the presence of such interlayers, in conjunction with other parameters of the hydrogeologic layers and of the species. The flow velocity in all cases of Problem 1 was $U = 0.1$ m/day, the system was saturated ($S = 1$), and the $z = 0$ boundary was kept at a constant concentration ($C_R = 1$).

8.1.1. Case 1-a: Effect of fracture offset (interlayers). This case involves the transport of the nonsorbing solute species ^3H in a layered fractured system with fracture offsets and various interlayer characteristics. Case 1-a involves three sub-cases: 1-a1, 1-a2 and 1-a3. The geometry of the reference Case 1-a1 of the layered fractured system is described in Table 5, while the hydraulic properties of the fractured layers are shown in Table 6. The three main layers (identified as Layers # 1,3 and 5 in Table 6) were fractured media (FM), while the interlayers (identified as Layers # 2 and 4) were considered to be fracture interlayers (FI, i.e., horizontal open fractures connecting the vertical fractures in the layers above and below).

The characteristics of Cases 1-a2 and 1-a3 are explained in Table 7, which shows only the differences from the base Case 1-a1. Thus, Cases 1-a2 and 1-a3 differ from Case 1-a1 in that the interlayers are porous interlayers (PI), i.e., the horizontal features connecting the fractured layers are either fractures filled with porous media or unfractured porous media. Flow and transport occurs through a porous medium with different transport behavior than in the FIsof Case 1-a1. The hydraulic properties of the porous media in the PIsin Cases 1-a2 and 1-a3 are the same as those of the porous matrix in the overlaying and underlying layers. The connecting PI in Cases 1-a2 and 1-a3 have a $b = 0.025$ m and 0.1 m, respectively. Note that in PI and PM layers there are no fractures and b represents the half-width of the flowing

portion of the matrix.

The results of the three subcases of Case 1-a are shown in Figure 7, which shows the fracture C_R . The presence of the interlayers in Figure 7 is marked by the vertical steps in the C_R profiles (caused by the fact that Figure 7 indicates the vertical coordinate z and not the length of the travel path. For the nonsorbing ^3H and at early times, the retardation caused by the presence of the FI is measurable, as compared to the case with aligned fractures (no interlayer, denoted by NI in Figure 7—included for comparison). This was expected because of the longer travel path in the case of FIs, which increase the amount of ^3H diffusing into the porous matrix and result in lower fracture concentrations. At the same early times, the retardation caused by the PIs can be substantial and increases with the half-width b of the PI. These results also conform with expectations because of the slower flow velocities in the porous media of the PI (as compared to those in the fractures of the FIs), which increase the residence time and diffusion into the porous matrix.

Figure 7 also shows that the effect of the interlayers keeps decreasing with time. This was expected in Case 1-a because the travel path increase caused by the interlayers is small (as the layer half-width X is only 0.25 m) and ^3H is nonsorbing (leaving diffusion into the matrix as the only mechanism removing the radionuclide from the flowing water). At $t = 10^4$ days, the presence of interlayers of any kind (FI vs. PI) has no effect on the concentration profile in the fractures.

8.1.2. Case 1-b: Combined effect of interlayers and matrix width of the fractured layers. This case involves three subcases: 1-b1, 1-b2 and 1-b3 (see Table 7). Cases 1-b1, 1-b2 and 1-b3 differed from Cases 1-a1, 1-a2 and 1-a3 in that $X = 2.5$ m instead of 0.25 m, thus substantially increasing the travel path and residence time of ^3H in the interlayers. This is expected to increase retardation, especially at early times.

Figure 8 confirms this expectation. At $t = 10^2$ days, the presence of the relatively fast flowing FI is sufficient to reduce C_R in the fracture by about four orders of magnitude. The effect is more pronounced in Case 1-b3 (PI with $b = 0.1$ m). The same pattern is observed

at $t = 10^3$ days, at which time the retardation in Case 1-b3 remains very substantial. This is caused by the reduction of the advective and dispersive components of transport (because velocity decreases as b increases) in addition to the reduction of the molecular diffusion component (due to the smaller ϕ and τ values in the filled fracture, see equation (2)). Remarkably, stronger retardation is observed in Case 1-b1 (FI) than in case 1-b2 (PI with $b = 0.025$ m). This is attributed to the larger solute mass in the PI, which is less affected by diffusion into the matrix (about the same in both cases). As in Case 1-a, the effect of the fracture offset (presence of interlayers) decreases with time.

The conclusion reached from these results is that the effect of fracture offsets (interlayers) increases with the matrix block size of the fractured layers. This is consistent with expectations because the travel path increases substantially in fractured system with large X , with a corresponding increase in residence time and diffusion into the matrix.

8.1.4. Case 1-c: Combined effect of interlayers and water saturation S of the fractured layers. This case involved two subcases: 1-c1 and 1-c2 (see Table 7). Cases 1-c1 differed from Case 1-a1 in that $S^m = 0.8$ and $S^f = 0.5$ instead of $S^m = S^f = 1$. Cases 1-c2 differed from Case 1-a3 in that $S^m = S^f = 0.8$ instead of $S^m = S^f = 1$. The effect of S is exhibited through its impact on the water velocity: a higher pore velocity V is needed to maintain the same U if S decreases. Thus, faster transport was expected in this case, with a corresponding decrease in the importance of the increased travel path caused by the fracture offset.

The results in Figure 9 confirm these expectations. Transport is faster than in Cases 1-a and 1-b, while the importance of the fracture offset (presence of interlayers) decreases in systems with the same water mass flow rate but with decreasing S_w .

8.2. Problem 2: Radioactive Solute Transport in a Complex Multi-Layered System

The complex geological system in Problem 2 is comprised of 14 layers and interlayers of fractured and porous media. The geometry and configuration of the system are described

in Table 8, and the rock properties and conditions are listed in Table 9. Linear equilibrium sorption is assumed, and the sorption coefficients of the various radionuclides in the fractures and in the matrix of the various layers (K_d^f and K_d^m , respectively) are listed in Table 10. The water velocity U at $z = 0$ as in Problem 1.

8.2.1. ^3H Transport. The fracture C_R profiles of the nonsorbing ^3H for both constant concentration (CC) and decaying (radioactively) concentration (DC) at the $z = 0$ boundary are shown in Figure 10, which includes observations at the following times: $t_1 = 10^4$ days, $t_2 = 5 \times 10^4$ days, $t_3 = 10^5$ days, $t_4 = 2.5 \times 10^5$ days and $t_5 = 5 \times 10^5$ days.

The various layers can be generally identified by a change in the C_R slope, while the interlayers are indicated by vertical sections of the C_R curves (as the abscissa is the z coordinate rather than the travel path). For a CC boundary, the C_R distribution reaches a steady state for $t \geq t_4$. As expected, the effect of the DC boundary is a C_R profile that is progressively lower than the one for a CC boundary, never reaches steady state, and is outside the C_R range ($< 10^{-9}$) for $t \geq t_4$.

8.2.2. ^{99}Tc Transport. ^{99}Tc (in its pertechnetate TcO_4^- speciation) is a non-sorbing radionuclide with a longer half life than ^3H (see Table 4). Two boundary conditions were considered in this case: a CC boundary and a piece-wise continuous (step) concentration (PC) boundary, i.e.,

$$C_R(z = 0) = \begin{cases} 1 & \text{for } t \leq 5 \times 10^4 \text{ days} \\ 0 & \text{for } t > 5 \times 10^4 \text{ days} \end{cases}$$

The C_R profiles in the fractures of the layered geologic system (at the same times as in the case of ^3H in Section 8.2.1) are shown in Figure 11. The effect of the longer half life is evident in the C_R profile for CC boundary, which indicates that ^{99}Tc advances much further in the formation than ^3H at the same times (the difference is due to radioactive decay), and does not appear to have reached steady state at $t = t_5$. The change in the boundary concentration over time in the PC boundary case results in C_R profiles that show a progressively larger (with time) ^{99}Tc -free zone near the boundary, while the C_R further into

the formation keeps decreasing and deviating from that for constant boundary concentration (with which it coincides fully or in part for $t \leq t_4$).

As indicated in the case of ^3H , the various layers and interlayers can be generally identified from changes in the C_R slope. Transport in fast flowing fractures (e.g., in the case of narrow fractures with large matrix blocks under a layer of wider fractures and narrow matrix blocks) can also be identified by a near-horizontal portion of the C_R profile.

8.2.3. ^{237}Np Transport. The C_R profile of the moderately sorbing ^{237}Np for a CC boundary is shown in Figure 12. The observation times are: $t_1 = 5 \times 10^4$ days, $t_2 = 10^5$ days, $t_3 = 5 \times 10^5$ days, $t_4 = 10^6$ days, $t_5 = 2.5 \times 10^6$ days and $t_6 = 5 \times 10^6$ days.

The slower transport of ^{237}Np (compared to that of ^{99}Tc) is caused by sorption and, to a far lesser extent, by increased diffusion into the matrix. Despite its longer half-life, the transport of ^{237}Np appears to be about an order of magnitude slower than that of ^{99}Tc , and does not appear to have reached steady state at $t = t_6$.

The C_R profiles along the x axis in the matrices of the various layers at $t = t_6$ are shown in Figure 13. The different shape of the curves is a function of their location (with respect to the $z = 0$ boundary and to the solute front) and of the transport properties of the matrix in the various layers.

8.3. Problem 3: Solute Transport of a Three-Member Radioactive Decay Chain in a Complex Multi-Layered System

Problem 3 describes the transport of the radioactive chain $^{239}\text{Pu} \rightarrow ^{235}\text{U} \rightarrow ^{231}\text{Pa}$ through the complex multilayered system described in Problem 2 (Tables 8 and 9). The sorption coefficients K_d^f and K_d^m of the ^{239}Pu parent in the various layers are listed in Table 10. The sorption coefficients of ^{235}U and ^{231}Pa in the fractures and in the matrix were assumed to be 5% and 50% of those for ^{239}Pu , respectively. C_R profiles of the three radionuclides were obtained at the following observation times: $t_1 = 10^5$ days, $t_2 = 10^6$ days, $t_3 = 10^7$ days, $t_4 = 10^8$ days, $t_5 = 10^9$ days, and $t_6 = 10^{10}$ days. Two boundary conditions were considered: a CC and a DC boundary.

8.3.1. ^{239}Pu Transport. Figure 14 shows the C_R profiles of ^{239}Pu in the fractures for constant boundary concentration and a decaying boundary concentration. There is no or little deviation of the two curves until $t = t_3$. The fracture C_R in DC case at $t = t_4$ is substantially lower than that of the CC case, and the C_R for a DC boundary is less than 10^{-9} for $t \geq t_5$.

An interesting observation is that, for a CC boundary, the ^{239}Pu front does not advance deep into the formation despite observation times orders of magnitude larger than those for the ^{237}Np transport. This is due to the very strong sorption of ^{239}Pu onto the matrix and fractures of the layers and, to a lesser extent, the shorter half life of ^{239}Pu (compared to that of ^{237}Np). Note that the C_R profile appears to have reached steady state at $t \geq t_5$.

In addition to the transport of the members of the chain, the transport of ^{239}Pu was studied separately, assuming a CC boundary and a $r \leq 1$ (see Equation (38) and the corresponding discussion). This describes a situation in which not all the contact area between fracture and matrix contributes to transport (e.g., because of a partially dry fracture which constitutes a discontinuity in the water phase). In this case, $r = S_w^f$ in the fractured layers and interlayers (FM or FI), and $r = 1$ elsewhere.

The effect of $r \leq 1$ in Figure 15 appears to have a substantial impact on transport, and results in a ^{239}Pu front that reaches much further (i.e., about three times deeper) in the geologic profile than that for $r = 1$. This is a direct consequence of a reduced area for ^{239}Pu diffusion from the fractures into the matrix, which leaves a larger amount of ^{239}Pu in the fractures where advection is fast and sorption relatively small (compared to the matrix). Thus, the transport of strongly sorbing radionuclides in fractured systems may be substantially influenced (enhanced) by partially dry fractures.

8.3.2. ^{235}U Transport. The fracture C_R profiles of ^{235}U for CC and DC boundaries and for $t \leq t_4$ are shown in Figure 16. The C_R of the DC solution always exceeding that from the CC solution, and significantly so (as imposed) in the vicinity of $z = 0$. A very significant observation is that, in either case, $C_R \simeq 1$ for $t \geq t_4$ in the top 120 m of the

domain. This is even more the case in Figure 17, which shows the C_R of ^{235}U for $t \geq 4$ and gives a more detailed picture of the C_R distribution near the value of 1. The results in Figures 16 and 17, in conjunction with the observations from Figure 14, indicate that for $t \geq t_4$, practically all of the radionuclide that advances deep into the formation is the ^{235}U daughter. The transport of ^{235}U is faster, the front reaches deeper, and $C_R \simeq 1$ because ^{235}U is generally weaker sorbing than ^{239}Pu and it has an extremely long half life. The obvious implication is that studies of ^{239}Pu transport cannot neglect the transport of the ^{235}U daughter, which is the dominant radionuclide at longer times.

Note from Figure 17 that, for $t = t_4$ and a DC boundary, $C_R > 1$, i.e., the ^{235}U concentration in the fractures exceeds the initial concentration of the ^{239}Pu parent at the $z = 0$ boundary. This is possible because the boundary (which introduces a radionuclide mixture composed of all the members of the ^{239}Pu decay chain) is now contributing a stream of almost 100% ^{235}U , which is added to the ^{235}U produced from the (almost complete) decay of ^{239}Pu already in the fractures and matrix of the system. As expected, the C_R from the CC solution at $t = t_4$ is lower than that from the DC solution (Figure 16). For $t > t_4$, the CC solutions exceed the DC solutions because the decay of the ^{235}U at the boundary is beginning to have an effect on the fracture distribution of C_R . This is particularly evident at $t = t_6$. Note that steady state is not reached (in either the CC or the DC boundary cases) even after $t_6 = 10^{10}$ days because of the extremely long half life of ^{235}U .

8.3.3. ^{231}Pa Transport. The fracture C_R profiles of ^{231}Pa for CC and DC boundaries are shown in Figure 18. The C_R levels of ^{231}Pa are quite low because of the very long half life of its ^{235}U parent, its own shorter half life, and its stronger tendency to sorb. The C_R increases with time for both DC and CC boundaries. The CC profile has always lower concentrations because there are all derived solely from the decay of ^{235}U (the boundary does not supply any additional ^{231}Pa in a CC regime). Note that in neither case, concentrations reach a steady state at about $t = t_4$.

9. Summary

In this paper, semianalytical solutions are developed for the problem of transport of radioactive or reactive solute tracers through a layered system of heterogeneous fractured media with misaligned fractures. The solutions allow any number and combination of fractured and/or porous layers that can vary in hydraulic and transport properties, fracture frequency, water saturation, fracture flow, and fracture-matrix interaction.

The tracer transport equations in the matrix account for (a) diffusion, (b) solute surface diffusion, (c) mass transfer between the mobile and immobile water fractions, (d) linear kinetic or equilibrium physical, chemical or combined solute sorption or colloid filtration, and (e) radioactive decay or first order chemical reactions. Any number of radioactive decay daughter products (or products of a linear, first-order reaction chain) can be tracked. The tracer transport equations in the fractures account for the same processes, as well as for advection and hydrodynamic dispersion. A wide array of boundary conditions (constant or time-variable, concentration or flux) can be accommodated.

Analytical solutions describing transport in the fracture and the matrix of each layer are first obtained in the Laplace space. These are impossible to invert analytically, and are numerically inverted by the method of *DeHoog et al.* [1982] to yield the solutions in time. These SA solutions are verified against analytical solutions of limiting cases of solute transport in fractured media. Additional verification is provided by comparisons against analytical solutions of transport in porous (unfractured) media.

The SA solutions are then tested in a series of hypothetical problems of increasing complexity. The effect of important parameters on the transport of ^3H , ^{237}Np and ^{239}Pu (and its daughters) is investigated in several test problems involving layered fractured geological systems. Fracture misalignment appears to significantly affect transport if water flow (and, consequently, transport) between the fractures of the overlaying and the underlying layers occurs through a porous connecting pathway.

The semianalytical solutions are computationally very efficient, requiring less than 10

seconds of execution time for the examples studied in this paper. The results of the test problems indicate that the semianalytical solutions can easily solve the problem of transport of parent and daughter radioactive species in multilayered heterogeneous systems under a variety of boundary conditions. Thus, they can provide a simple and effective tool to predict radionuclide transport in subsurface environments involving saturated/unsaturated flow through variably fractured media (such as transport from the potential repository through the fractured rock layers in the UZ of Yucca Mountain to the water table). While such predictions are quasi 2-D and do not account for spatial variability and flow effects in the 3-D continuum of the subsurface (such as perched water bodies, flow diversion and flow focusing), they can provide bounding estimates that bracket the true solution.

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Appendix A: The H_ν and $T_{\nu,\kappa}$ Coefficients

For Case 2 ($X \rightarrow \infty$), the $H_\nu \equiv H_\nu^e$ of the first five members of a radioactive or reactive chain ($\nu = 1, \dots, 5$) are

$$H_1^e = \widehat{C}_1^f$$

$$H_2^e = \widehat{C}_2^f - A_{21} \widehat{C}_1^f$$

$$H_3^e = \widehat{C}_3^f - A_{32} \widehat{C}_2^f + A_{21}(A_{32} - A_{31}) \widehat{C}_1^f$$

$$\begin{aligned}
H_4^e &= \widehat{C}_4^f - A_{43} \widehat{C}_3^f + A_{32}(A_{43} - A_{42}) \widehat{C}_2^f \\
&\quad - A_{21} [A_{32}(A_{43} - A_{42}) - A_{31}(A_{43} - A_{41})] \widehat{C}_1^f \\
H_5^e &= \widehat{C}_5^f - A_{54} \widehat{C}_4^f + A_{43}(A_{54} - A_{53}) \widehat{C}_3^f \\
&\quad - A_{32} [A_{43}(A_{54} - A_{53}) - A_{42}(A_{54} - A_{52})] \widehat{C}_2^f \\
&\quad + A_{21} \{ A_{32} [A_{43}(A_{54} - A_{53}) - A_{42}(A_{54} - A_{52})] \\
&\quad - A_{31} [A_{43}(A_{54} - A_{53}) - A_{41}(A_{54} - A_{51})] \} \widehat{C}_1^f
\end{aligned}$$

in which the m superscript of the A factors (equation (52)) are omitted for simplicity. The terms $T_{\nu,\kappa}$ in equation (53) can be easily identified by inspection. By following the emerging pattern, the development of the expressions for H_ν for $\nu > 5$ is tedious but straightforward.

The $H_\nu \equiv H_\nu^c$ expressions (corresponding to Case 1) are entirely analogous, and are derived by dividing H_ν^e by $\cosh(\theta_\nu X)$. For example, for $\nu = 2$,

$$H_2^c = \frac{\widehat{C}_2^f}{\cosh(\theta_2 X)} - \frac{A_{21} \widehat{C}_1^f}{\cosh(\theta_2 X)}.$$

Appendix B: The W_ν and $\gamma_{\nu,\kappa}$ Coefficients

For Case 2 ($X \rightarrow \infty$), the $W_\nu \equiv W_\nu^e$ of the first 5 members of a radioactive or reactive chain ($\nu = 1, \dots, 5$) are

$$\begin{aligned}
W_1^e &= \theta_1 C_1^f \\
W_2^e &= \theta_2 C_2^f + A_{21} (\theta_1 - \theta_2) \widehat{C}_1^f \\
W_3^e &= \theta_3 C_3^f + A_{32} (\theta_2 - \theta_3) \widehat{C}_2^f + A_{21} [A_{31} \theta_1 - A_{32} \theta_2 + (A_{32} - A_{31}) \theta_3] \widehat{C}_1^f \\
W_4^e &= \theta_4 C_4^f + A_{43} (\theta_3 - \theta_4) \widehat{C}_3^f + A_{32} [A_{42} \theta_2 - A_{43} \theta_3 + (A_{43} - A_{42}) \theta_4] \widehat{C}_2^f \\
&\quad + A_{21} \{ A_{31} A_{41} \theta_1 - A_{32} A_{42} \theta_2 + A_{43} (A_{32} - A_{31}) \theta_3 \\
&\quad - [A_{32} (A_{43} - A_{42}) - A_{31} (A_{43} - A_{41})] \theta_4 \} \widehat{C}_1^f
\end{aligned}$$

$$\begin{aligned}
W_5^e &= \theta_5 C_5^f + A_{54} (\theta_4 - \theta_5) \widehat{C}_4^f + A_{43} [A_{53} \theta_3 - A_{54} \theta_4 + (A_{54} - A_{53}) \theta_5] \widehat{C}_3^f \\
&+ A_{32} \{ A_{42} A_{52} \theta_2 - A_{43} A_{53} \theta_3 + A_{54} (A_{43} - A_{42}) \theta_4 \\
&\quad - [A_{43} (A_{54} - A_{53}) - A_{42} (A_{54} - A_{52})] \theta_5 \} \widehat{C}_2^f \\
&+ A_{21} \left\{ A_{31} A_{41} A_{51} \theta_1 - A_{32} A_{42} A_{52} \theta_2 + A_{43} A_{53} (A_{32} - A_{31}) \theta_3 \right. \\
&\quad - A_{54} [A_{32} (A_{43} - A_{42}) - A_{31} (A_{43} - A_{41})] \theta_4 \\
&\quad + [A_{32} [A_{43} (A_{54} - A_{53}) - A_{42} (A_{54} - A_{52})] \\
&\quad \quad \left. - A_{31} [A_{43} (A_{54} - A_{53}) - A_{41} (A_{54} - A_{51})] \right\} \theta_5 \widehat{C}_1^f,
\end{aligned}$$

in which the m superscript of the A factors (equation (52)) are omitted for simplicity.

We obtain W_ν^c for Case 1 by replacing θ_ν by $\theta_\nu \tanh(\theta_\nu X)$ in W_ν^e . Thus, for $\nu = 2$ and Case 1,

$$W_2^c = \theta_2 \tanh(\theta_2 X) C_2^f + A_{21} [\theta_1 \tanh(\theta_1 X) - \theta_2 \tanh(\theta_2 X)] \widehat{C}_1^f$$

The terms $\gamma_{\nu,\kappa}$ are easy to obtain from (59) and the W_ν^c, W_ν^e expressions by inspection.

Extension for $\nu > 5$ follows the same pattern.

Appendix C: The $B_{\nu,\kappa}^\pm$ Coefficients

The $B_{\nu,\kappa}^\pm$ coefficients of up to the first 5 members of a radioactive or reactive chain ($\nu = 1, \dots, 5, \kappa = 1, \dots, \nu - 1$) are given by the following general expressions:

$$B_{\nu,\nu-1}^\pm = \gamma_{\nu,\nu-1} f^q - G_\nu^f$$

$$B_{\nu,\nu-2}^\pm = (\gamma_{\nu,\nu-1} f^q - G_\nu^f) A_{\nu-1,\nu-2}^\pm + \gamma_{\nu,\nu-2} f^q$$

$$B_{\nu,\nu-3}^\pm = (\gamma_{\nu,\nu-1} f^q - G_\nu^f) A_{\nu-1,\nu-3}^\pm + (\gamma_{\nu,\nu-2} A_{\nu-2,\nu-3}^\pm + \gamma_{\nu,\nu-3}) f^q$$

$$B_{\nu,\nu-4}^\pm = (\gamma_{\nu,\nu-1} f^q - G_\nu^f) A_{\nu-1,\nu-4}^\pm + (\gamma_{\nu,\nu-2} A_{\nu-2,\nu-4}^\pm + \gamma_{\nu,\nu-3} A_{\nu-3,\nu-4}^\pm + \gamma_{\nu,\nu-4}) f^q$$

The coefficients A^\pm needed for the computation of B^\pm are obtained from equation (63). All other terms are as discussed in Section 5.2. Extension for $\nu > 5$ follows the same pattern.

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Table 1. Input parameters in Test FS1

Parameters	Values
Water saturation S	1
PM grain density ρ	2600 kg/m ³
D_0	1.6×10^{-9} m ² /s
Fracture aperture $2b$	10^{-4} m
Fracture S	1
Fracture ϕ	1
Fracture τ	1
Fracture K_d	0 m
Longitudinal dispersivity α_L in the fracture	0.1 m
Fracture flow velocity V	0.1 m/day
Matrix block width $2X$	0.5 m
Matrix S	1
Matrix ϕ	0.01
Matrix τ	0.1
Matrix K_d	0 m ³ /kg
Radionuclide $T_{1/2}$	12.35 years (tritium)
Z_1, Z_2, Z_3 (for $N = 3$)	1 m, 9 m, ∞

Table 2. Input parameters in Tests PS1 to PS4

Parameters	Values
ρ	2600 kg/m ³
D_0	5×10^{-2} m ² /day
S	1
ϕ	0.1
τ	1
V	0.1 m/day
K_d (Tests PS1 and PS4)	0 m ³ /kg
K_d (Tests PS2 and PS3)	4.2735042×10^{-5} m ³ /kg
$T_{1/2}$ (Tests PS1 and PS2)	∞ (stable isotopes)
$T_{1/2}$ (Tests PS3 and PS4)	100 days
Z_1, Z_2, Z_3 (for $N = 3$)	10 m, 10 m, ∞

Table 3. Input parameters in Test PS5

Parameters	Values
ρ	2600 kg/m ³
D_0	1000 m ² /year
S	1
ϕ	0.3
τ	1
V	100 m/year
K_d for ²³⁴ U	1.64819 m ³ /kg
K_d for ²³⁰ Th	8.24159 m ³ /kg
K_d for ²²⁶ Ra	8.22528×10^{-2} m ³ /kg
$T_{1/2}$ of ²³⁴ U	2.45×10^5 years
$T_{1/2}$ of ²³⁰ Th	7.54×10^4 years
$T_{1/2}$ of ²²⁶ Ra	1.60×10^3 years
Z_1, Z_2, Z_3 (for $N = 3$)	50 m, 150 m, ∞

Table 4. Radionuclide properties used in the transport simulations of Section 8

Radionuclide	D_0 (m ² /s)	$\lambda = \frac{\ln 2}{T_{1/2}}$ (1/s)
³ H	1.60×10^{-9}	1.778×10^{-9}
⁹⁹ Tc	4.55×10^{-10}	1.031×10^{-13}
²³⁷ Np	7.12×10^{-10}	1.026×10^{-14}
²³⁹ Pu	6.08×10^{-10}	9.114×10^{-13}
²³⁵ U	6.08×10^{-10}	3.1023×10^{-17}
²³¹ Pa	6.08×10^{-10}	6.7583×10^{-13}

Table 5. Layer geometry in Case 1-a of Problem 1

Layer #	Type	Parameter	Value
1	FM	Z	5 m
		X	0.25 m
		b	5×10^{-5} m
2	FI	b	5×10^{-5} m
3	FM	Z	10 m
		X	0.25 m
		b	5×10^{-5} m
4	FI	b	5×10^{-5} m
5	FM	Z	∞ m
		X	0.25 m
		b	5×10^{-5} m

Table 6. Properties in Case 1-a1 of Problem 1

Layer #	Parameters	Values
1,3,5	α_L	0.1 m
	ϕ^m	0.01
	$\tau^m = \tau_p^m = \tau_i^m = \tau_s^m$	0.1
	ϕ^f	1
	$\tau^f = \tau_p^f = \tau_i^f = \tau_s^f$	1
	$K_i^m = K_i^f$	1
	$S_w^m = S_w^f$	1
2,4	α_L	0.1 m
	ϕ^m	0.01
	$\tau^m = \tau_p^m = \tau_i^m = \tau_s^m$	0.1
	ϕ^f	1
	$\tau^f = \tau_p^f = \tau_i^f = \tau_s^f$	1
	$K_i^m = K_i^f$	1
	$S^m = S^f$	1

Table 7. Parameter variations in the various cases of Problem 1

Case #	Layer #	Type	Parameter	Value
1-a2	2,4	PI	b	0.025 m
1-a3	2,4	PI	b	0.10 m
1-b1	1,3,5	FI	X	2.5 m
	2,4	FI	Z	2.5 m
1-b2	1,3,5	FI	X	2.5 m
	2,4	PI	Z	2.5 m
			b	0.025 m
1-b3	1,3,5	FI	X	2.5 m
	2,4	PI	Z	2.5 m
			b	0.10 m
1-c1	All	FM, FI	S_w^m	0.8
			S_w^f	0.5
1-c2	All	FM, PI	$S_w^m = S_w^f$	0.8
	2,4	PI	b	0.1 m

Table 8. Layer geometry in Problem 2

Layer #	Type	Z (m)	X (m)	b (m)
1	FM	10	0.5	10^{-4}
2	PI			5×10^{-2}
3	FM	10	0.25	5×10^{-5}
4	PI			2.5×10^{-2}
5	FM	10	3	2×10^{-4}
6	PM	5		
7	FM	15	0.1	2×10^{-4}
8	PI			10^{-1}
9	FM	10	4	2×10^{-5}
10	FI			2×10^{-5}
11	FM	20	1	5×10^{-5}
12	PM	5		
13	FM	30	6	8×10^{-5}
14	PM	∞		

Table 9. Rock properties in Problem 2

Layer #	ϕ^m	τ^m	S_w^m	ϕ^f	τ^f	S_w^f
1	0.15	0.5	0.7	1	1	0.2
2	0.3	0.3	1	0.3	0.3	0.4
3	0.1	0.4	0.6	1	1	0.15
4	0.35	0.3	1	0.35	0.3	0.3
5	0.05	0.5	0.8	1	1	0.1
6	0.35	0.8	0.9	0.35	0.8	0.9
7	0.025	0.2	0.9	1	1	0.1
8	0.2	0.3	0.9	0.2	0.3	0.4
9	0.01	0.2	0.95	1	1	0.05
10	0.01	0.2	0.95	1	1	0.05
11	0.05	0.15	0.95	1	1	0.05
12	0.1	0.1	0.9	0.2	0.1	0.9
13	0.05	0.1	1	1	1	1
14	0.1	0.1	1	0.1	0.1	1

Table 10. Transfer coefficients in Problem 2

Layer #	³ H or ⁹⁹ Tc		²³⁷ Np		²³⁹ Pu	
	K_d^m (*)	K_d^f (†)	K_d^m (*)	K_d^f (†)	K_d^m (*)	K_d^f (†)
1	0	0	6×10^{-4}	3×10^{-8}	6×10^{-2}	3×10^{-6}
2	0	0	8×10^{-4}	8×10^{-4}	8×10^{-2}	8×10^{-2}
3	0	0	7×10^{-4}	3.5×10^{-8}	7×10^{-2}	3.5×10^{-6}
4	0	0	8×10^{-4}	8×10^{-4}	8×10^{-2}	8×10^{-2}
5	0	0	8×10^{-4}	4×10^{-8}	8×10^{-2}	4×10^{-6}
6	0	0	10^{-4}	10^{-4}	10^{-2}	10^{-2}
7	0	0	10^{-3}	5×10^{-7}	10^{-1}	5×10^{-5}
8	0	0	8×10^{-4}	8×10^{-4}	8×10^{-2}	8×10^{-2}
9	0	0	5×10^{-4}	2.5×10^{-8}	5×10^{-2}	2.5×10^{-6}
10	0	0	5×10^{-4}	2.5×10^{-8}	5×10^{-2}	2.5×10^{-6}
11	0	0	9×10^{-4}	4.5×10^{-8}	9×10^{-2}	4.5×10^{-6}
12	0	0	10^{-3}	10^{-3}	10^{-1}	10^{-1}
13	0	0	6×10^{-4}	3×10^{-8}	6×10^{-2}	3×10^{-6}
14	0	0	7×10^{-4}	7×10^{-4}	7×10^{-2}	7×10^{-2}

(*): in m³/kg, (†): in m

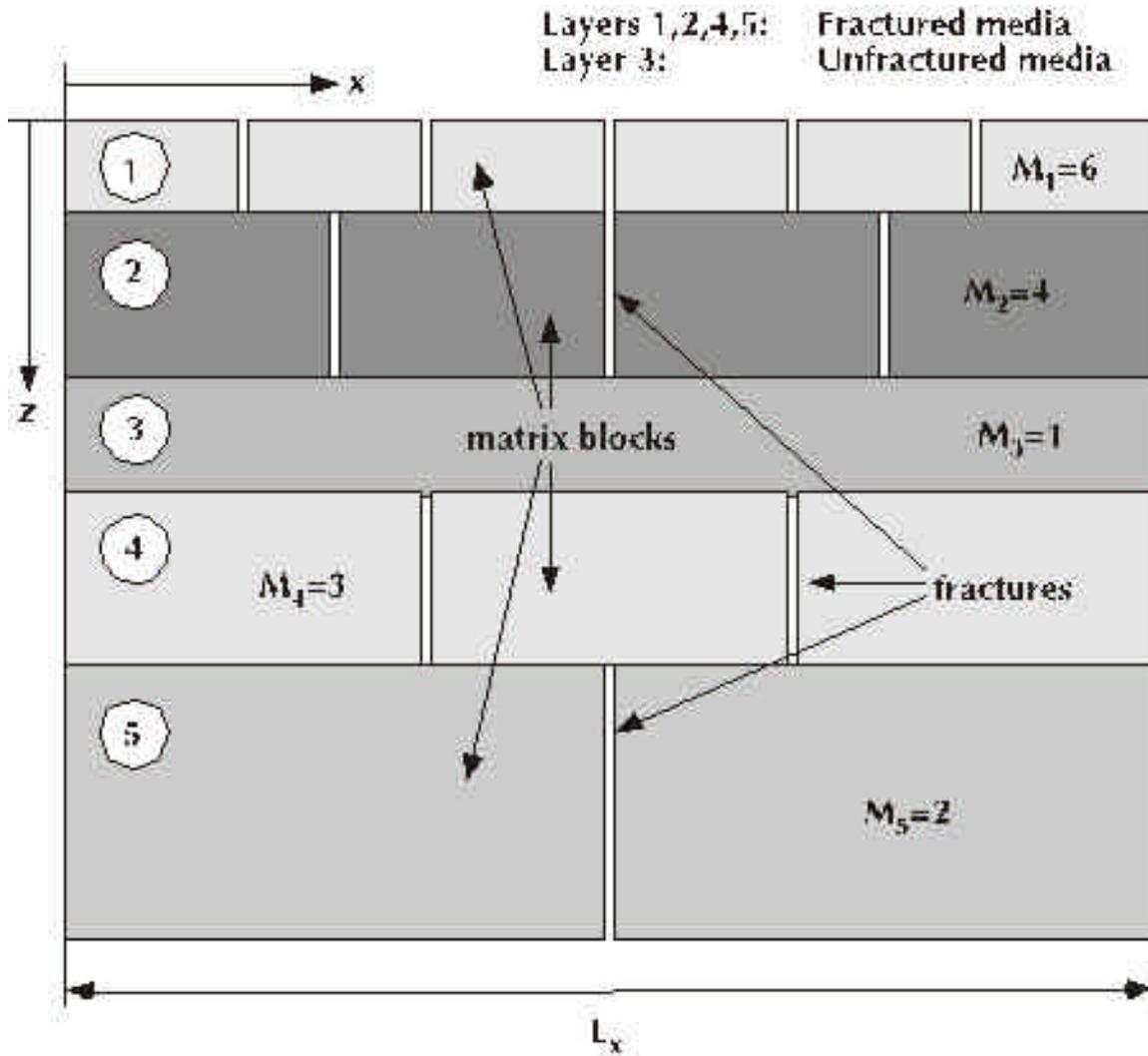


Figure 1. A variably-fractured layered geologic system.

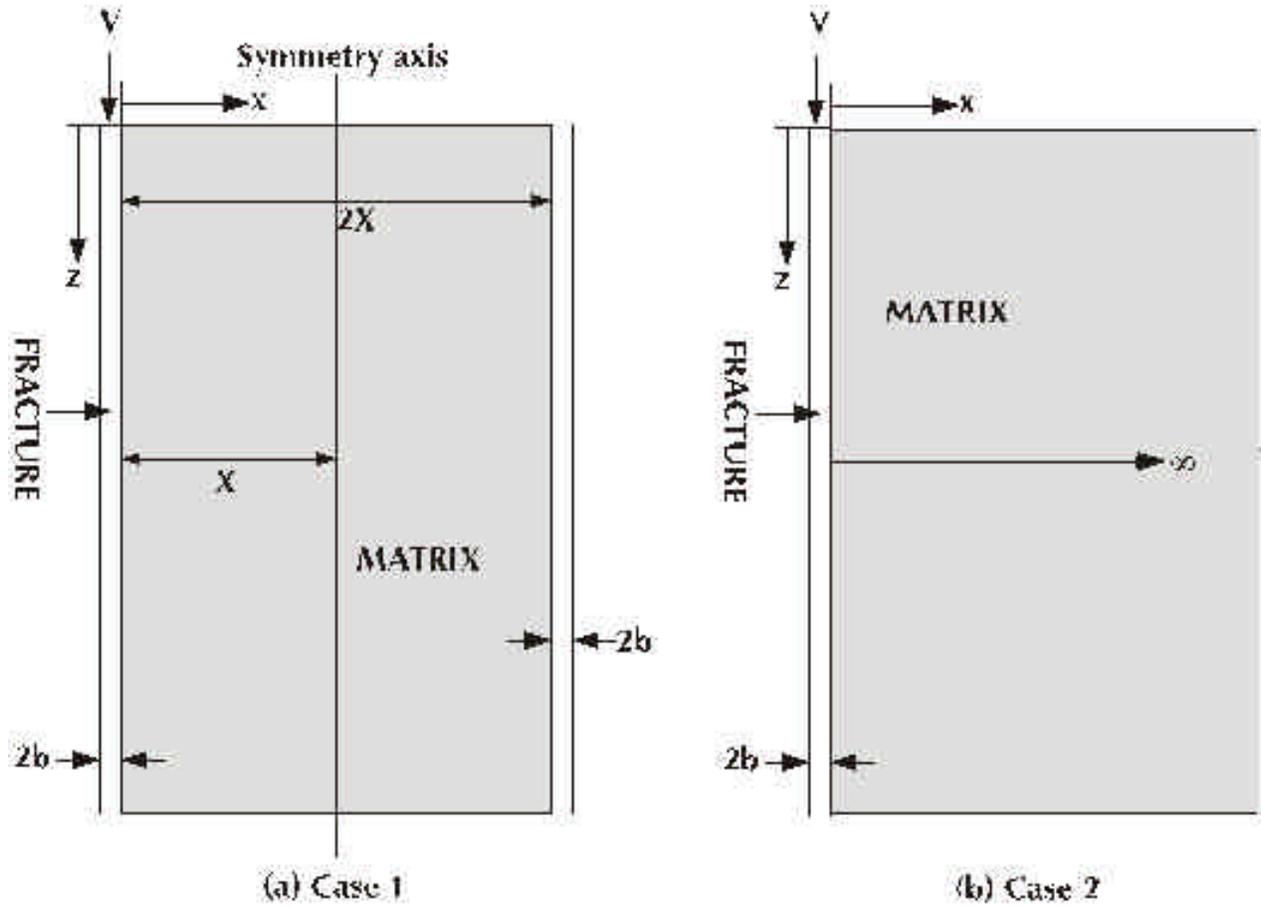


Figure 2. Fracture-matrix configurations and important parameters in Cases 1 and 2.

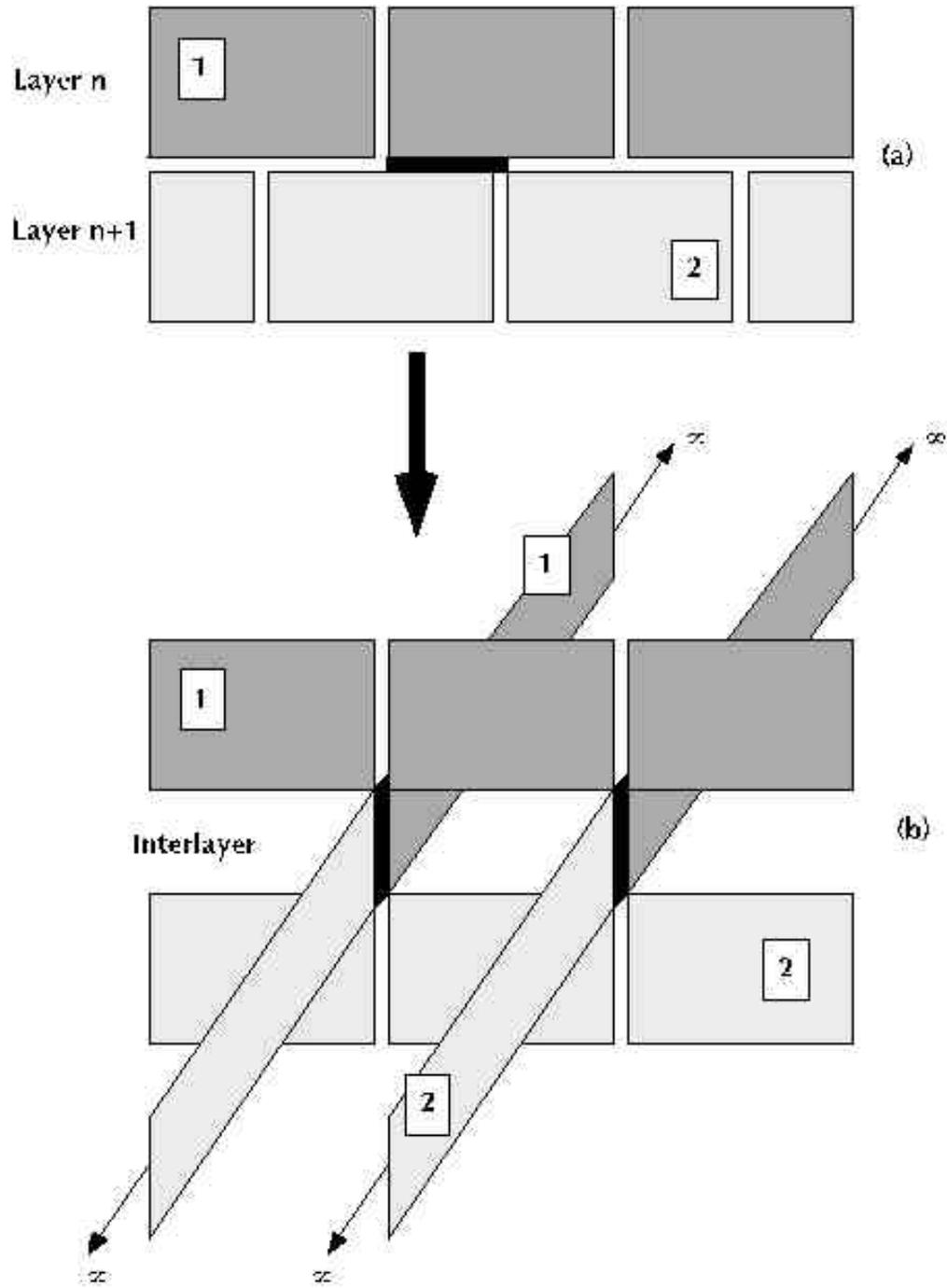


Figure 3. A graphic representation of the concept of interlayer describing the effects of fracture misalignment. The properties of layers n and $n+1$ are denoted by 1 and 2, respectively.

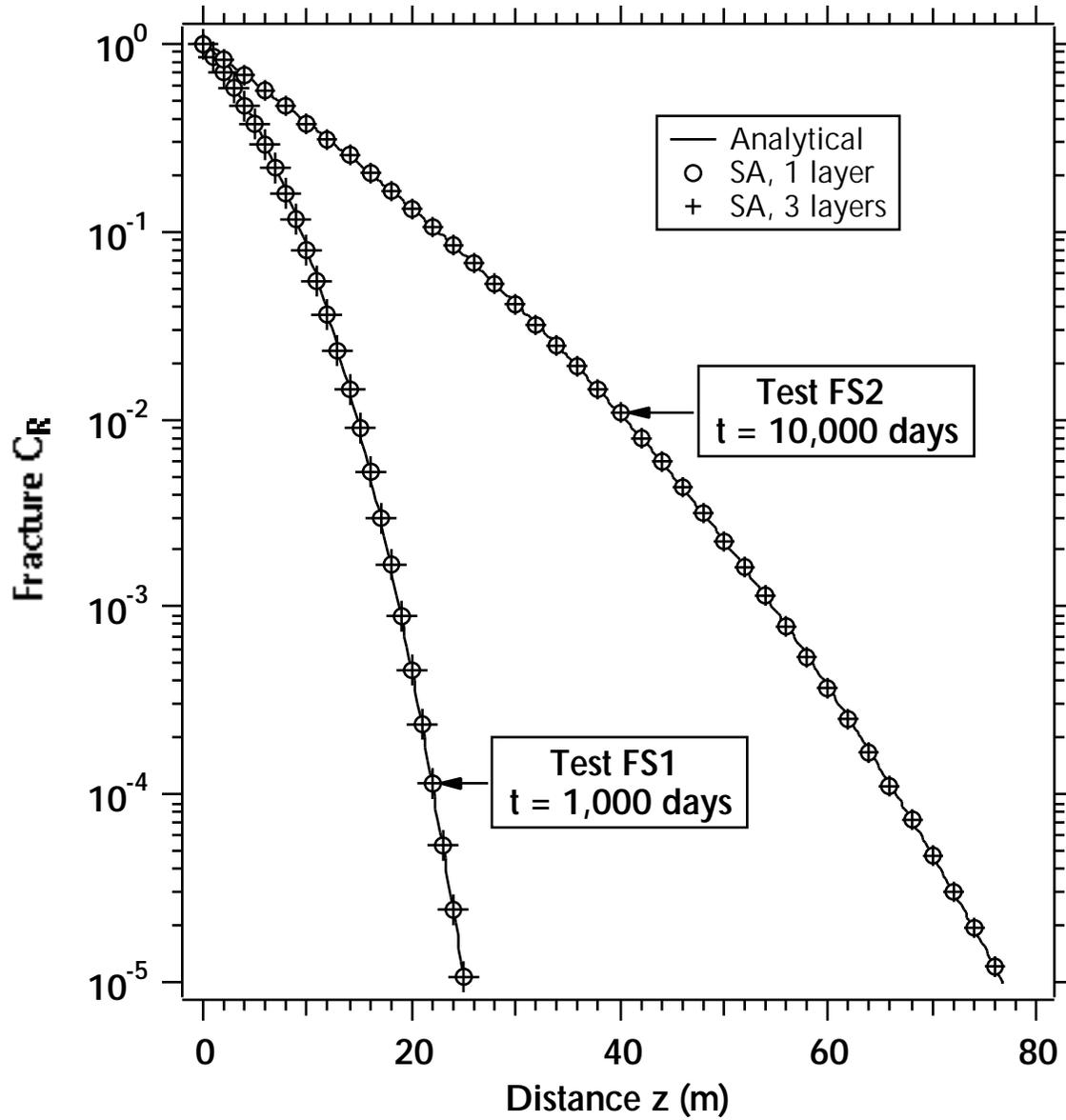


Figure 4. Comparison of the semianalytical (SA) solutions from FRACL to the analytical solution of radioactive solute transport in fractured media in Tests FS1 and FS2.

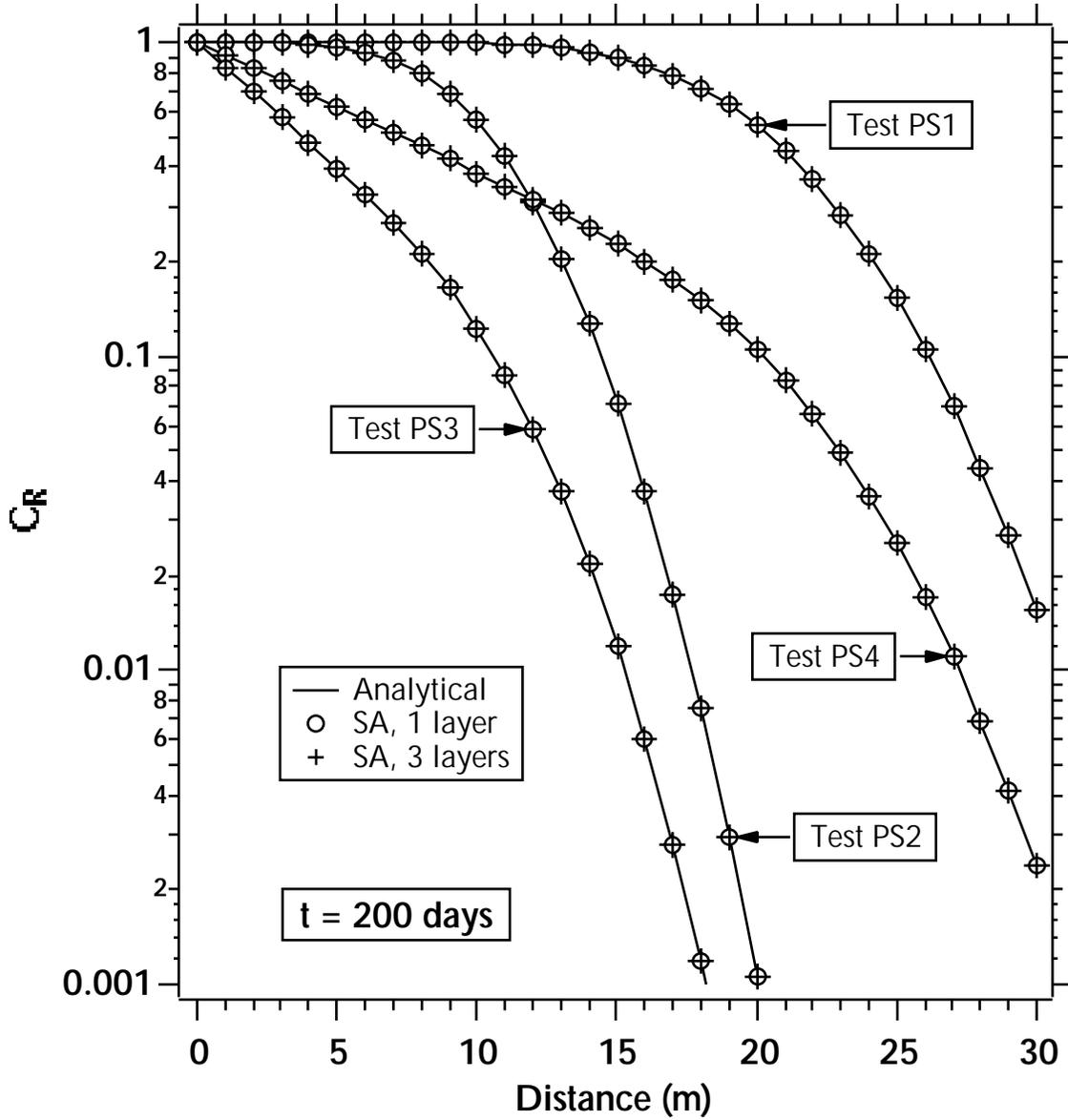


Figure 5. Comparison of the SA solutions to the analytical solutions of solute transport in porous media in Tests PS1 to PS4.

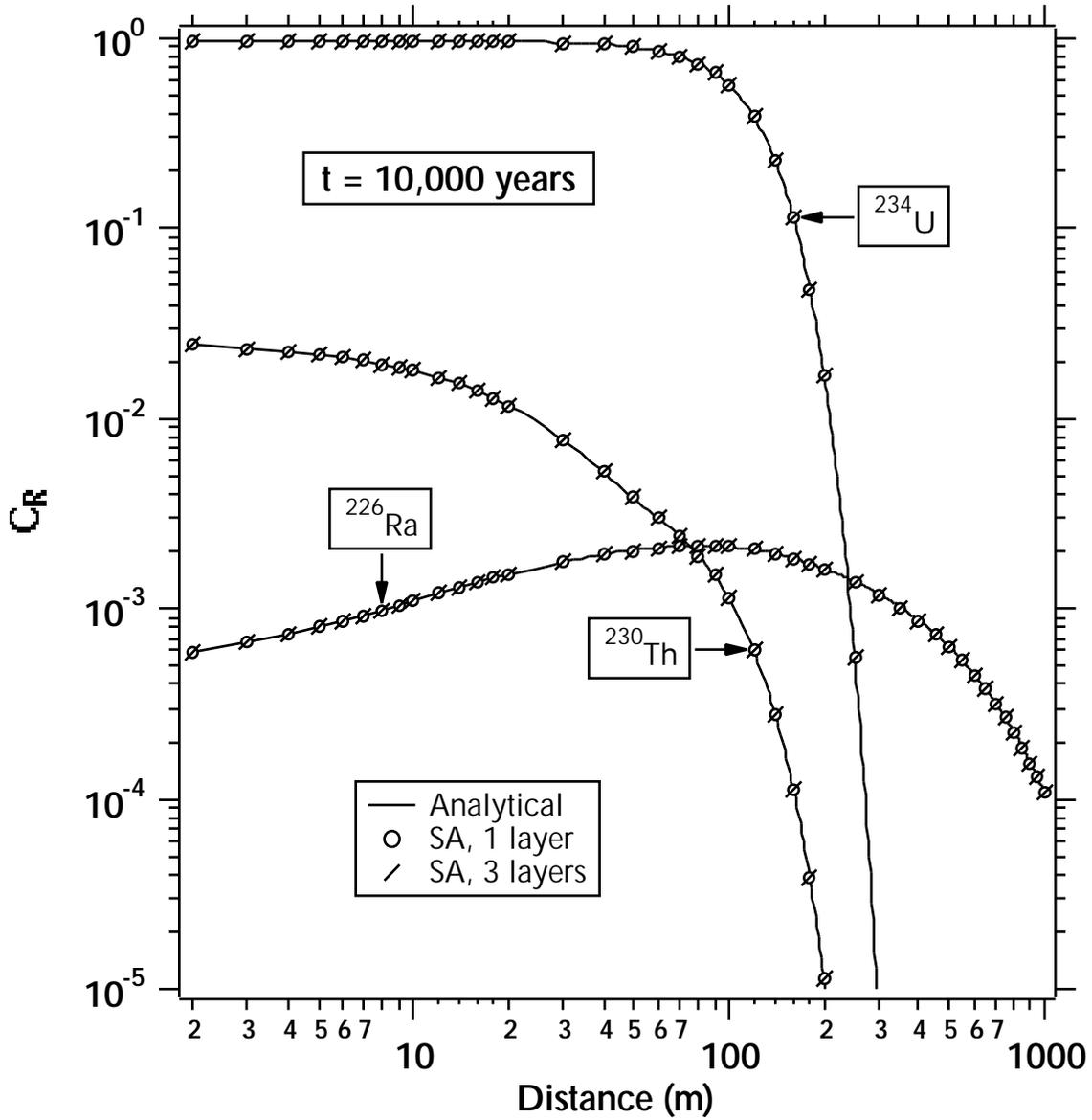


Figure 6. Comparison of the SA solutions from FRACL to the analytical solutions of solute transport of the radioactive chain $^{234}\text{U} \rightarrow ^{230}\text{Th} \rightarrow ^{226}\text{Ra}$ in porous media in Test PS5.

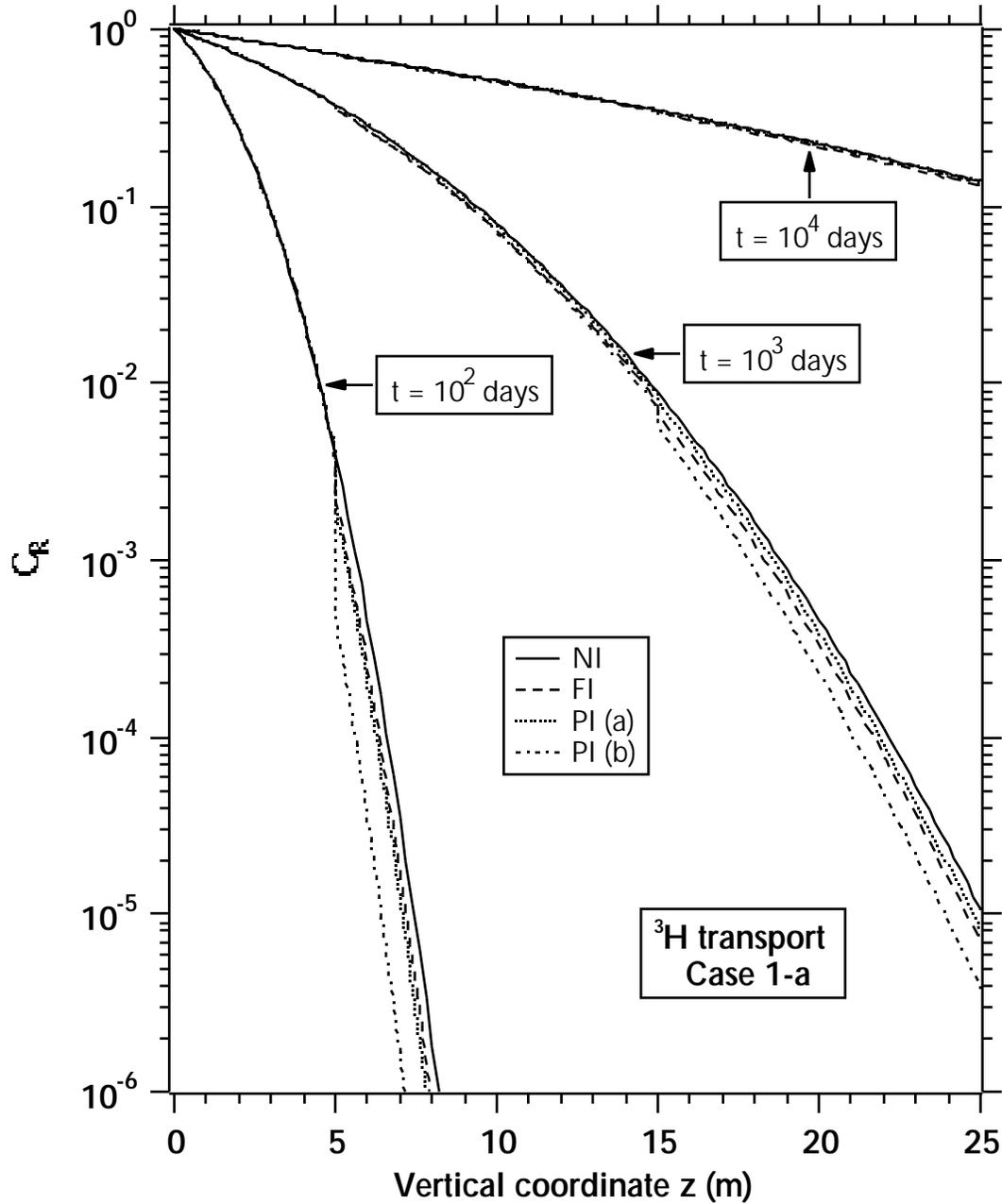


Figure 7. Effect of fracture offset (presence of interlayers) on the transport of ^3H through the layered fractured system of Case 1-a (NI: no interlayer, FI: fracture interlayer, PI(a): porous interlayer with $b = 0.025$ m, PI(b): porous interlayer with $b = 0.1$ m).

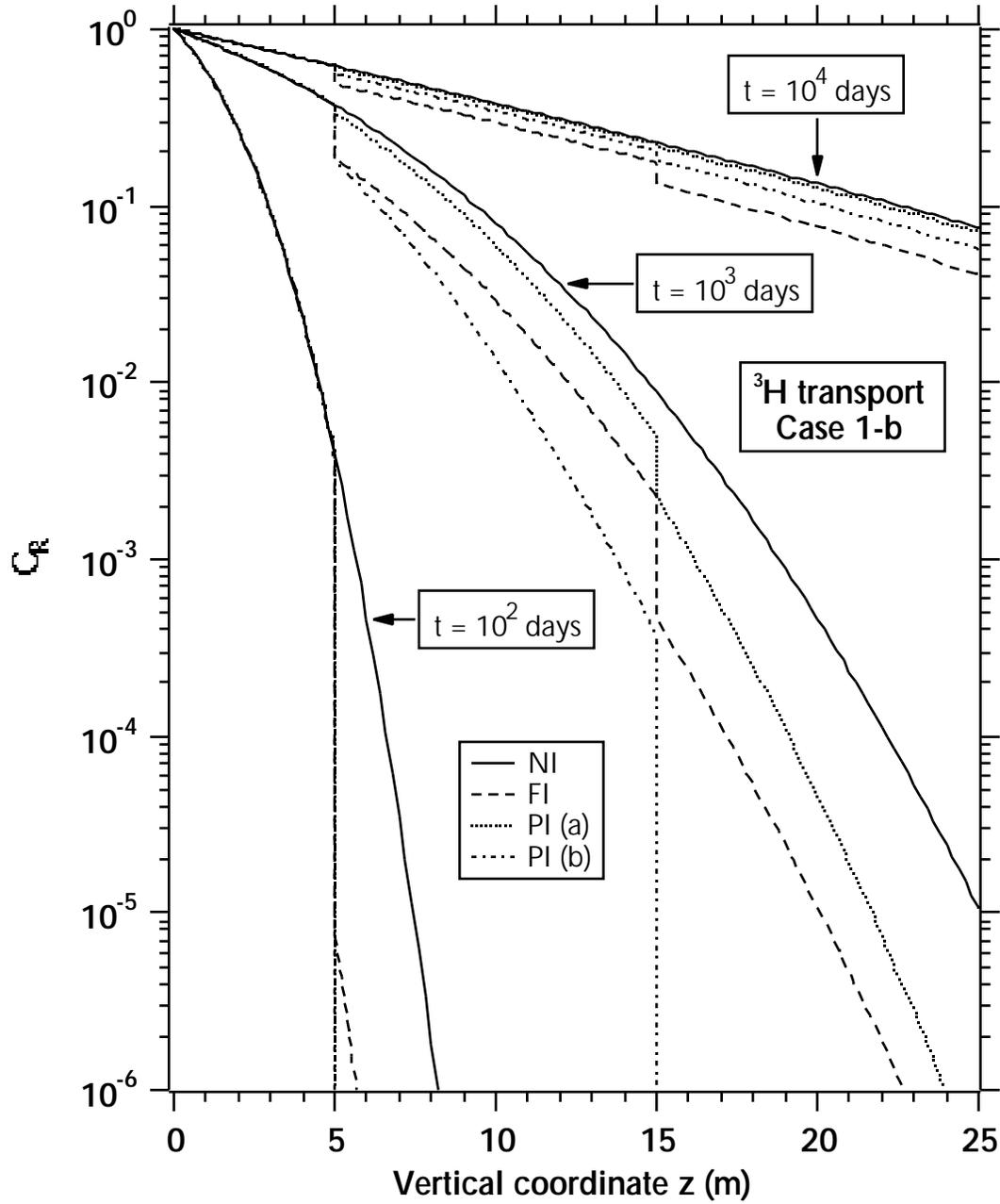


Figure 8. Combined effect of increased X and fracture offset (presence of interlayers) on the transport of ^3H through the layered fractured system of Case 1-b (nomenclature as in Figure 7).

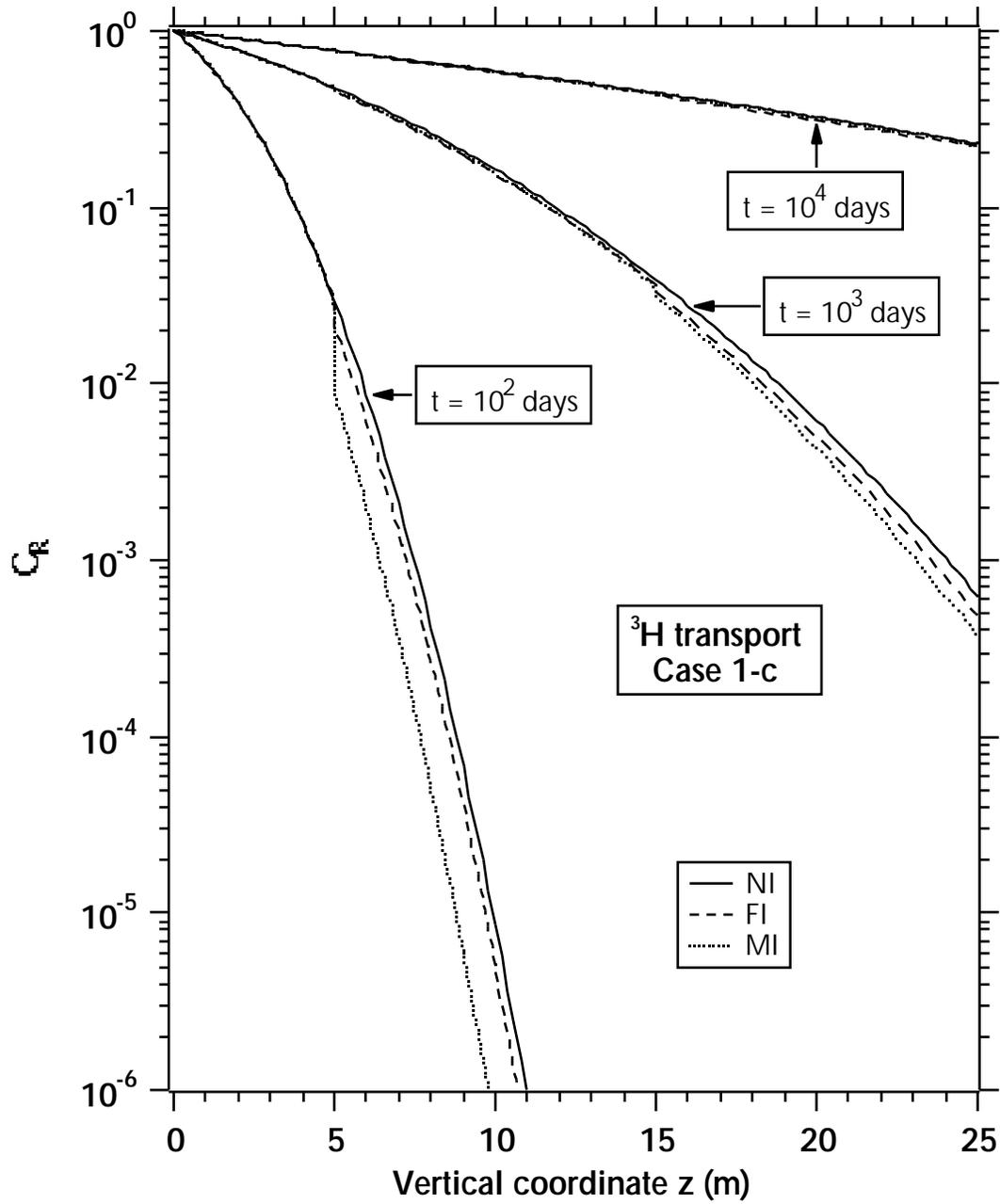


Figure 9. Combined effect of water saturation S and fracture offset on the transport of ${}^3\text{H}$ through the layered fractured system of Case 1-c (nomenclature as in Figure 7).

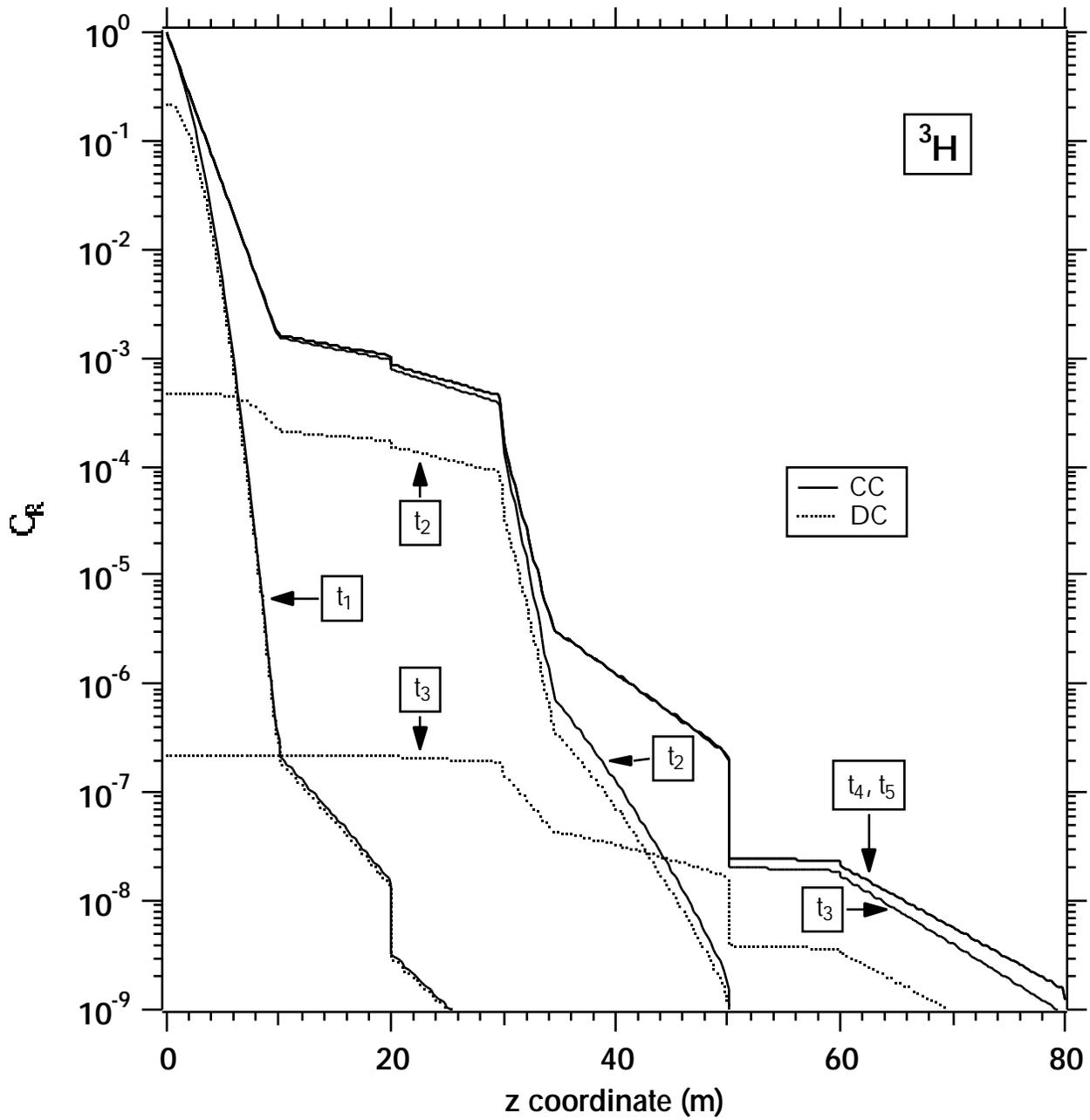


Figure 10. Fracture C_R profiles of ^3H in the complex geological system of Problem 2 (CC: constant concentration boundary, DC: decaying concentration boundary).

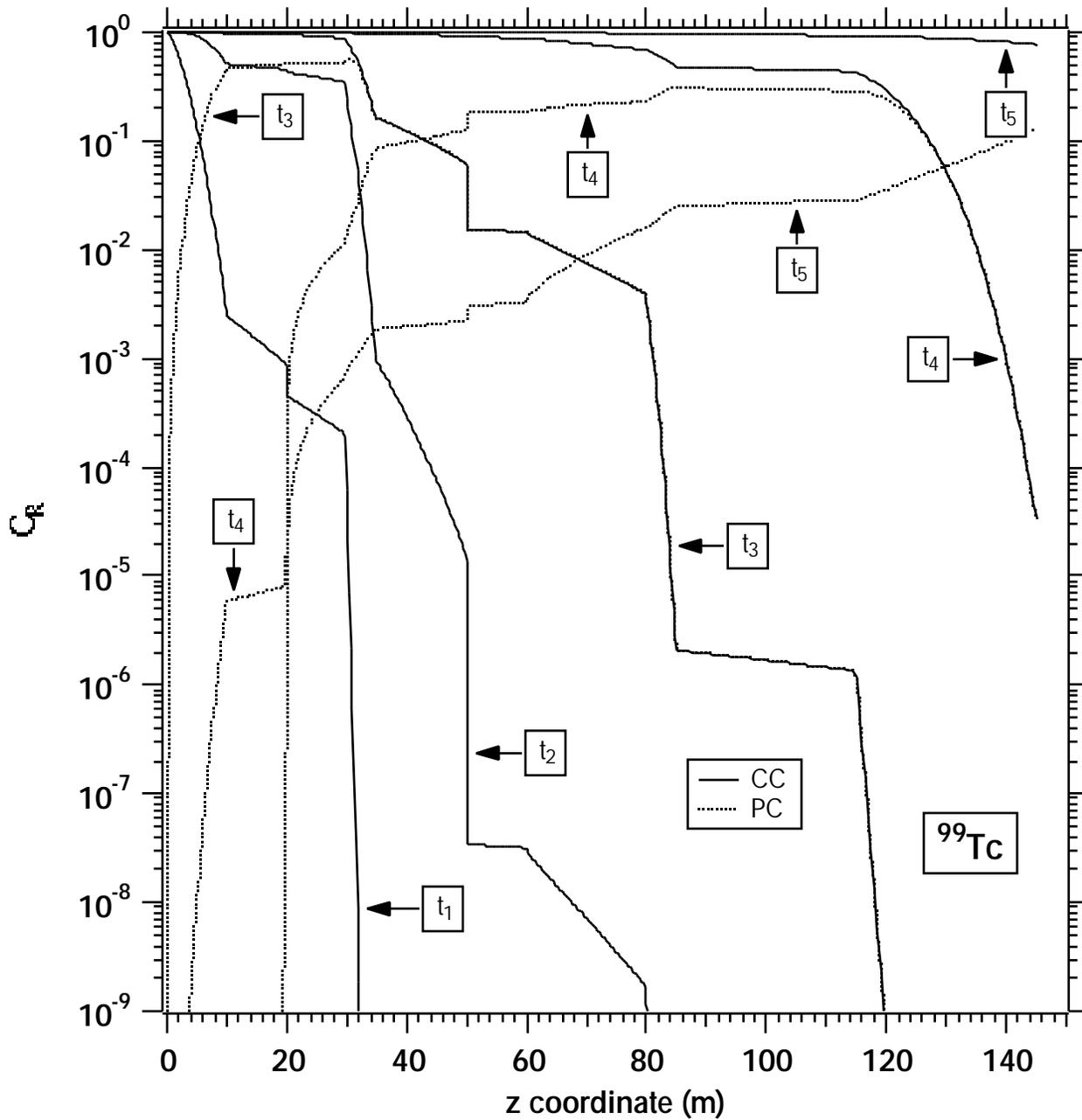


Figure 11. Fracture C_R profiles of ^{99}Tc in the geological system of Problem 2 (PC: pulse concentration boundary).

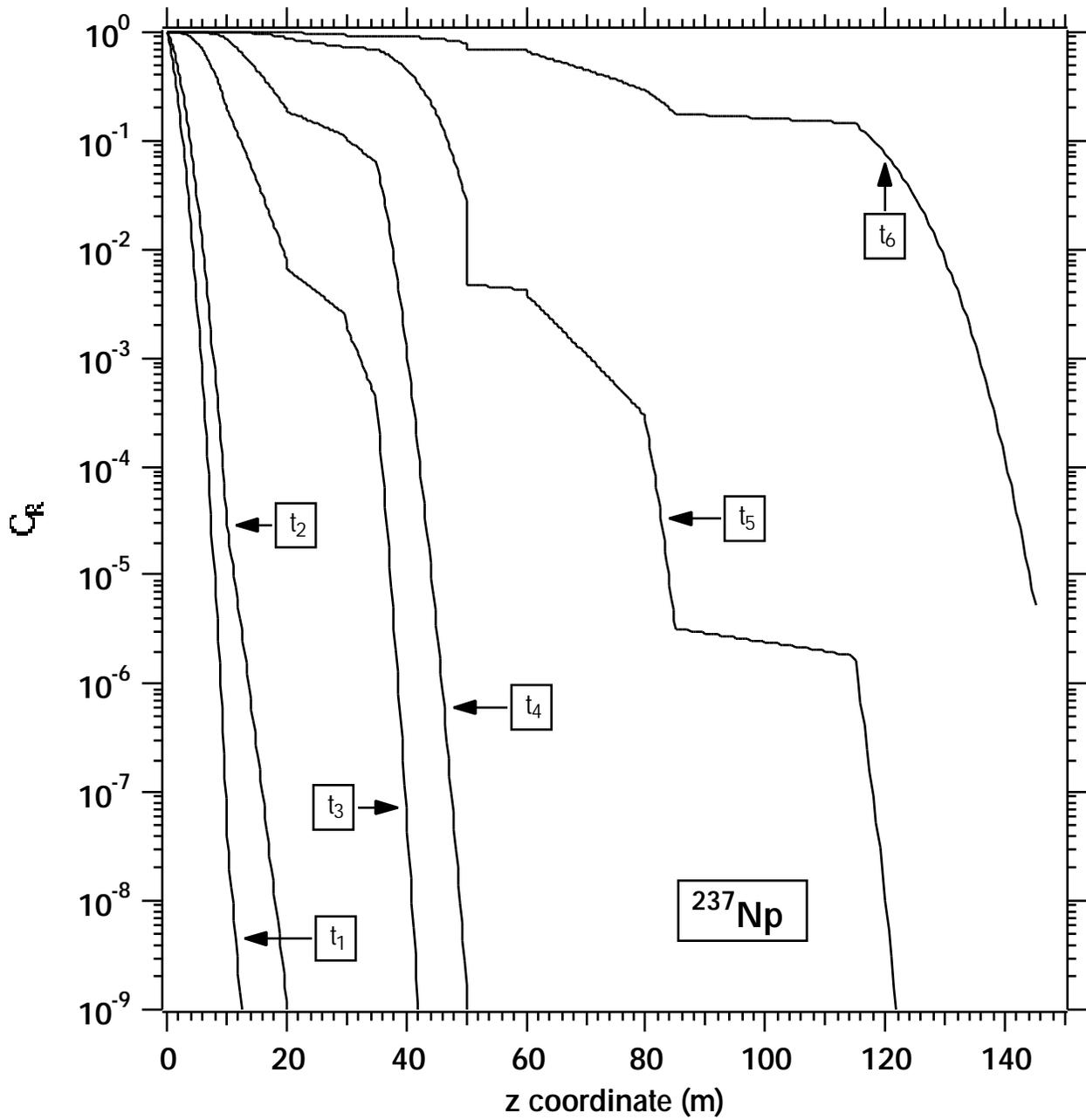


Figure 12. Fracture C_R profiles of ^{237}Np in the geological system of Problem 2.

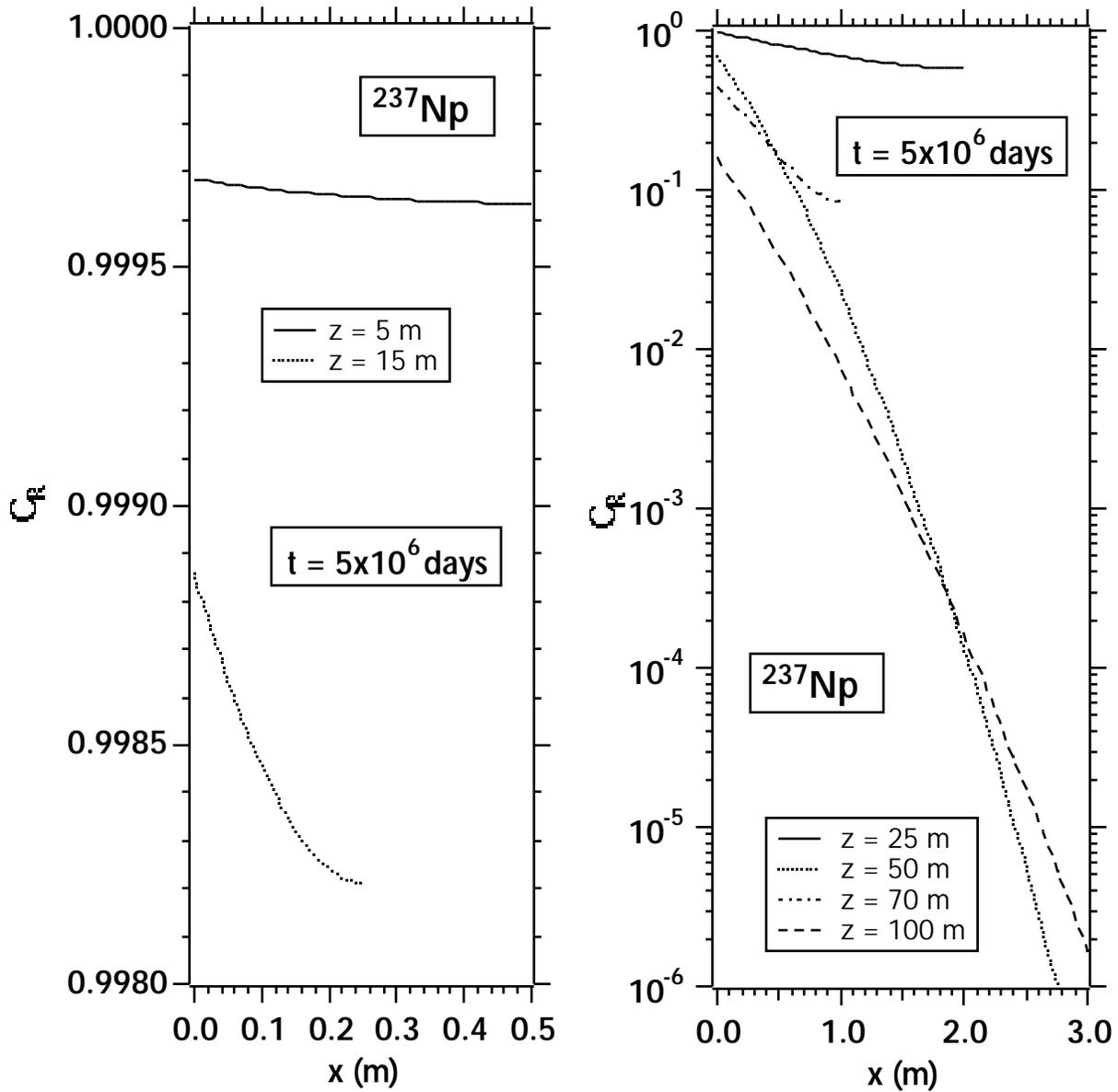


Figure 13. Matrix C_R profiles of ^{237}Np at different elevations in the geological system of Problem 2.

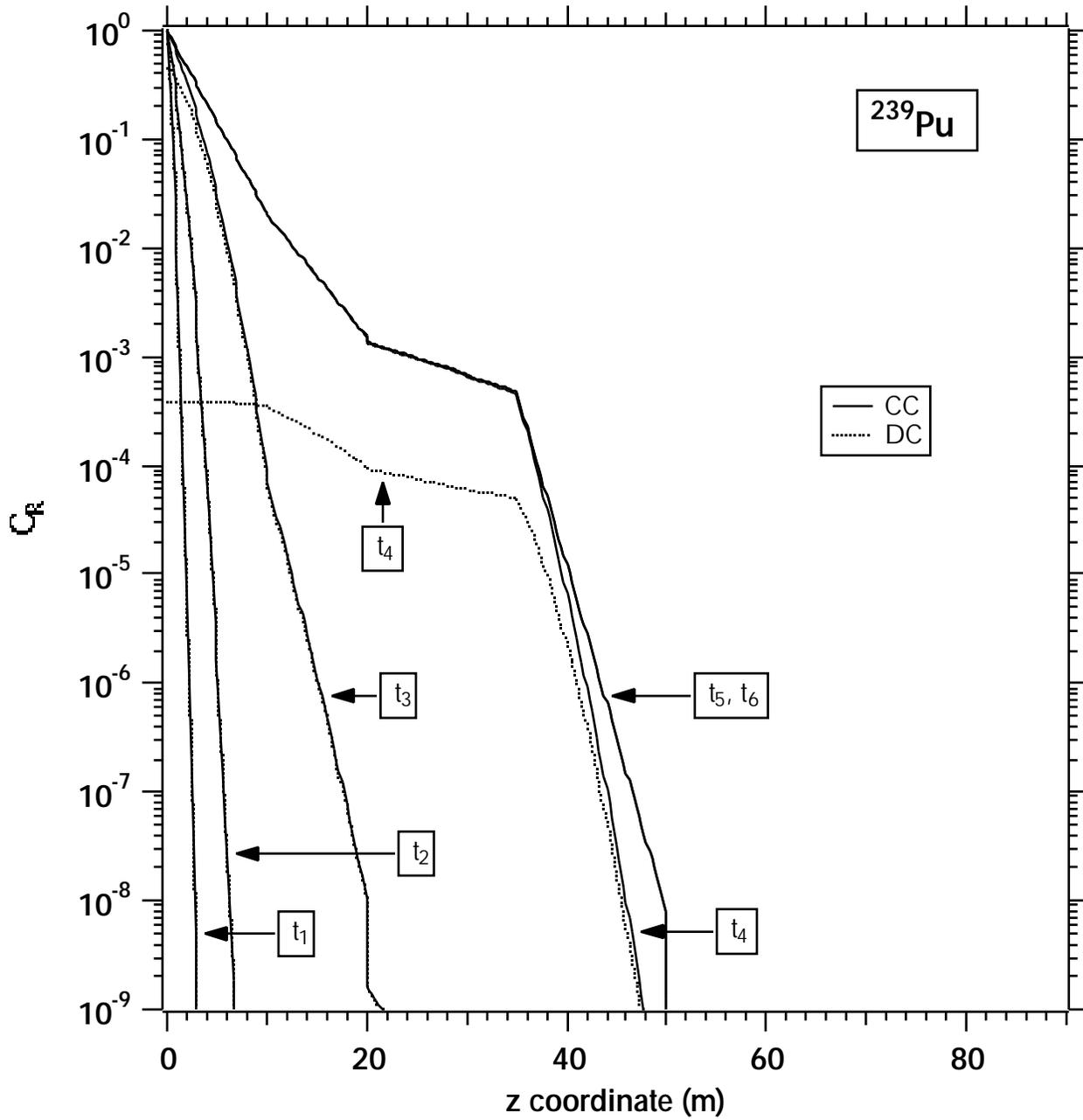


Figure 14. Fracture C_R profiles of ^{239}Pu in the geological system of Problem 3.

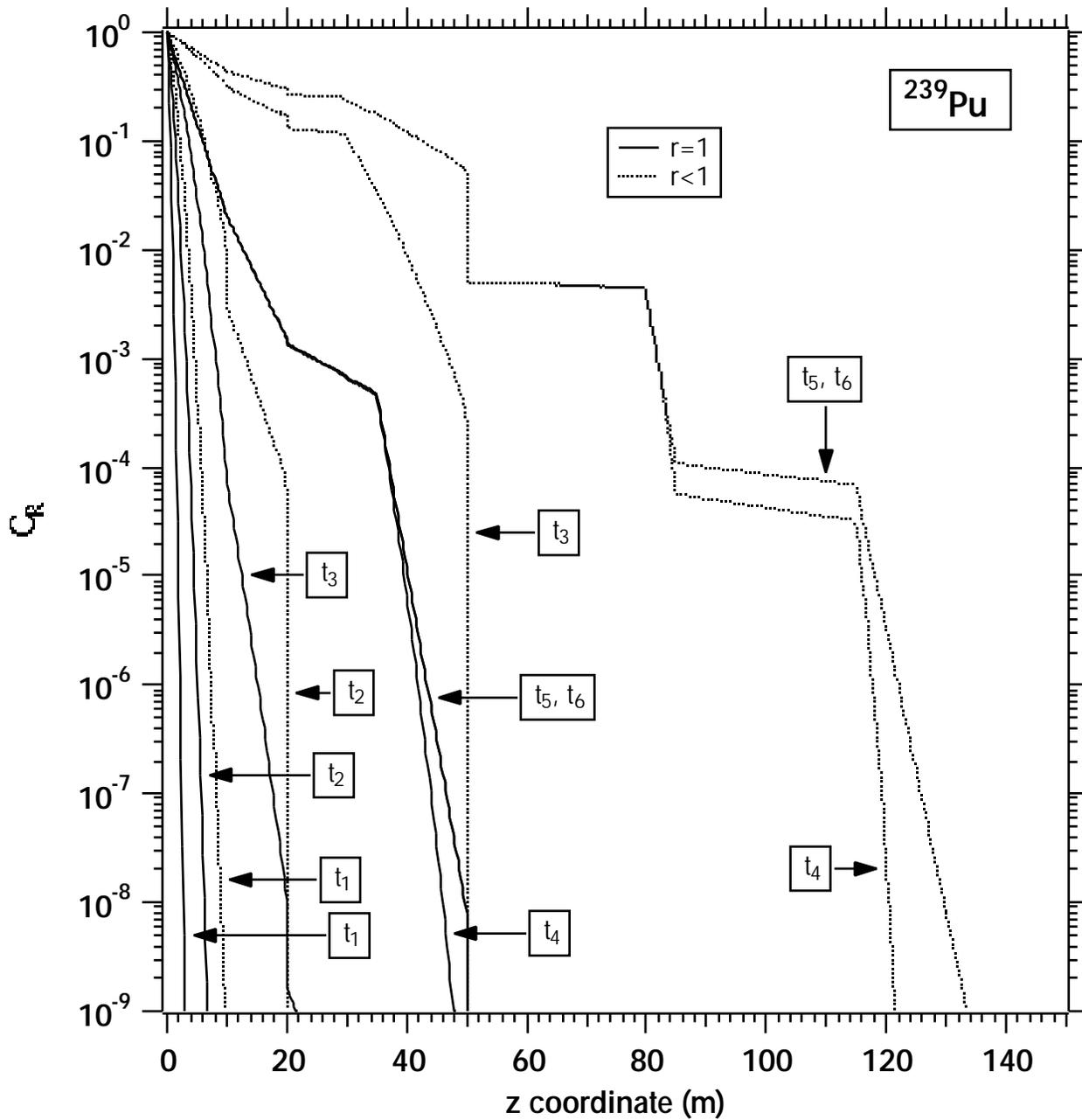


Figure 15. Effect of $r < 1$ on the fracture C_R profiles of ^{239}Pu in the geological system of Problem 3.

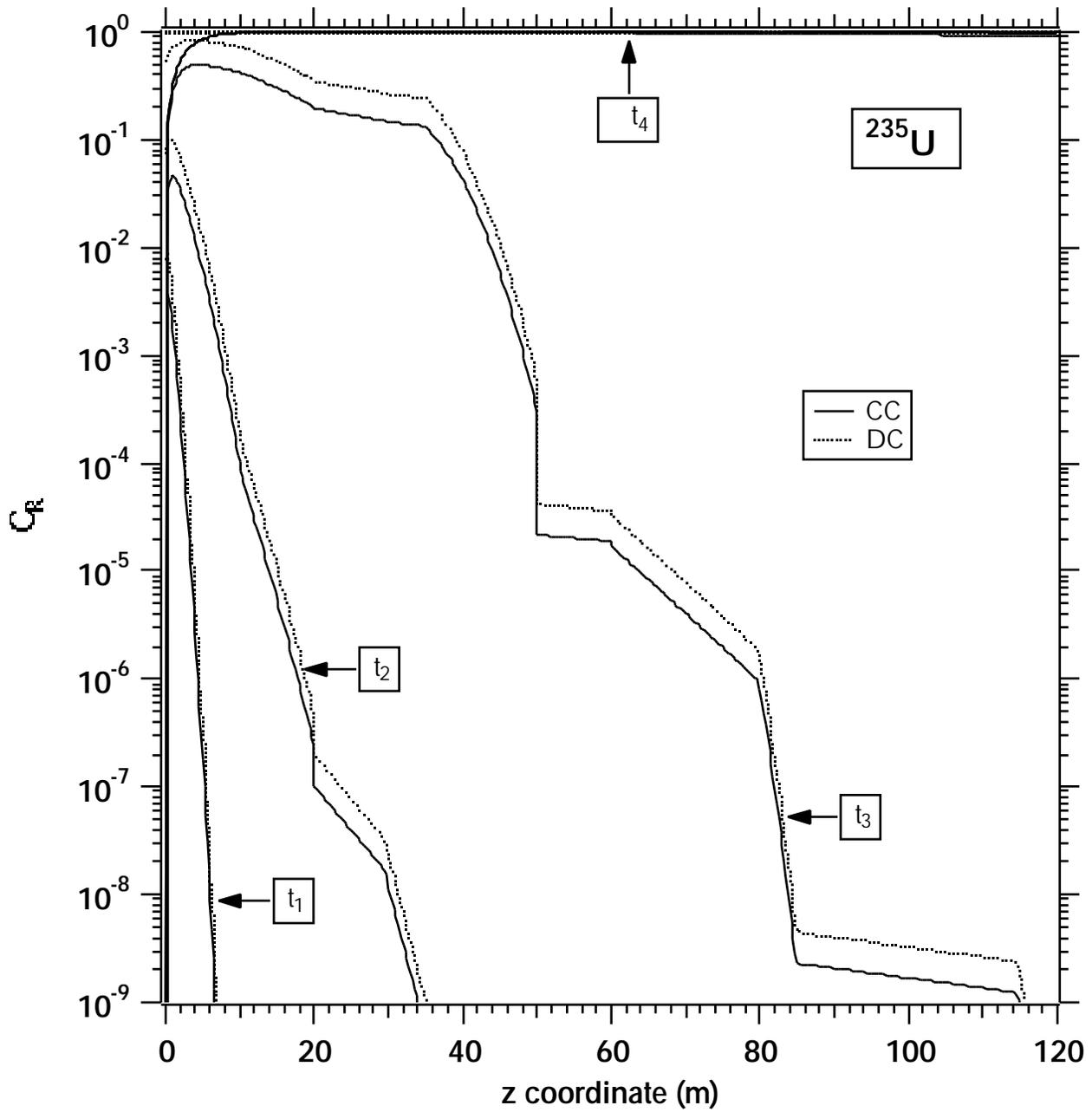


Figure 16. Fracture C_R profiles of ^{235}U in the geological system of Problem 3 for $t \leq 10^8$ days.

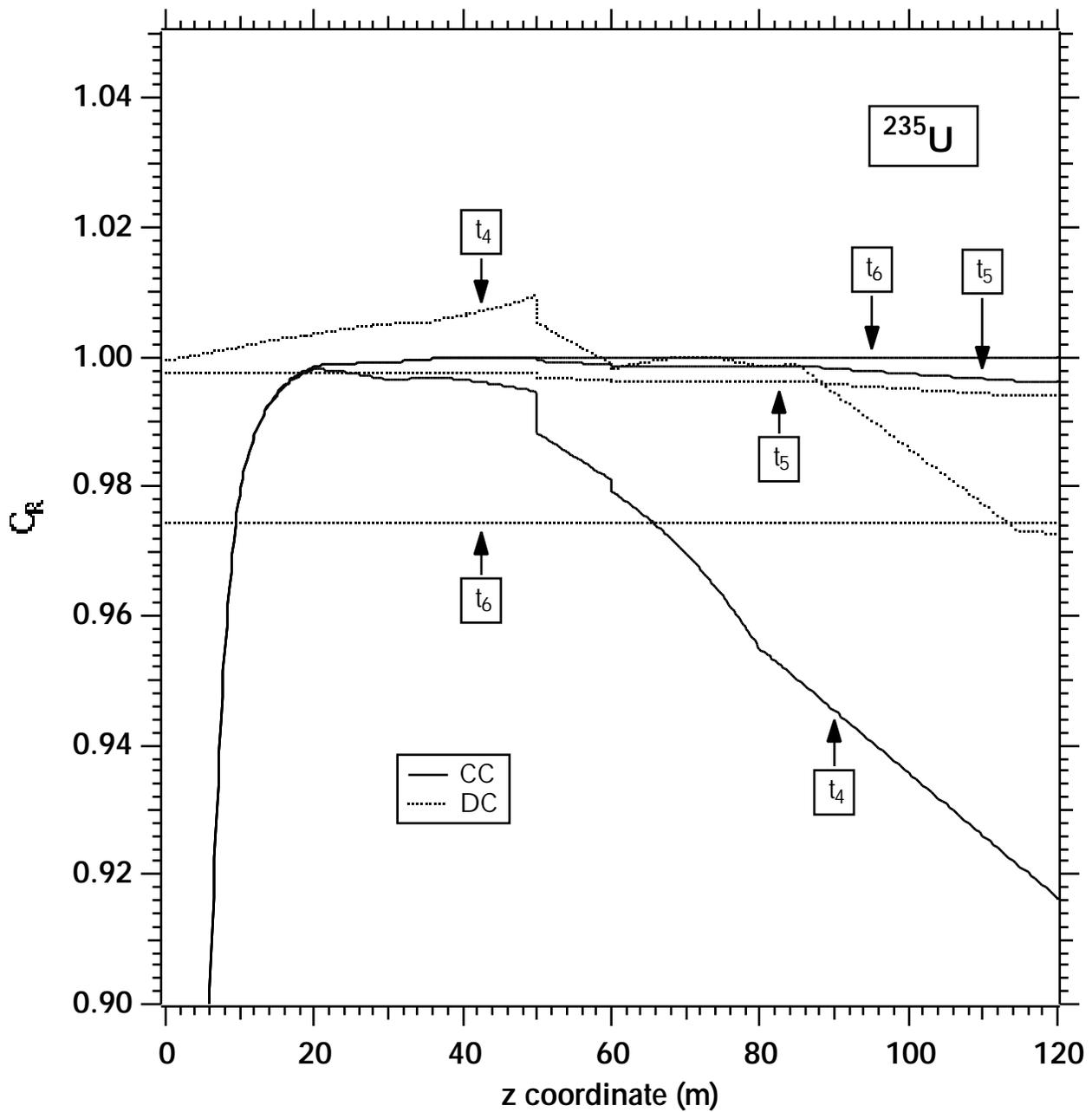


Figure 17. Fracture C_R profiles of ^{235}U in the geological system of Problem 3 for $t \geq 10^8$ days.

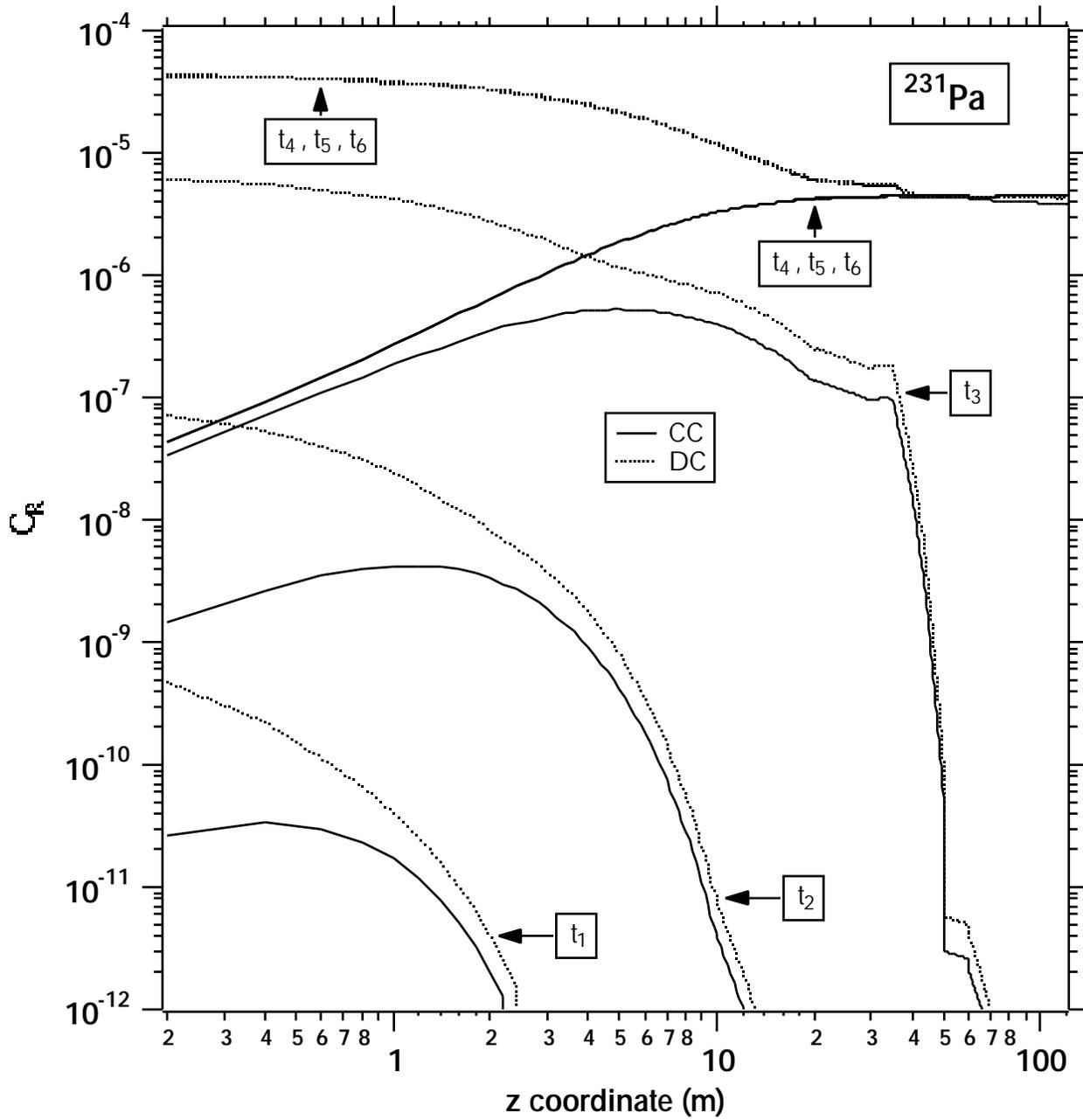


Figure 18. Fracture C_R profiles of ^{231}Pa in the geological system of Problem 3.