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**UCB-NE-107 User's Manual**

W.W.-L. Lee

March 1989

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LBL-26672  
c.1

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LBL-26672  
UCB-NE-4132  
UC-70

# **UCB-NE-107**

## **User's Manual**

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**March 1989**

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## INTRODUCTION

The purpose of this manual is to provide users of UCB-NE-107 with the information necessary to use UCB-NE-107 effectively.

## DESCRIPTION OF THE CODE

UCB-NE-107<sup>1</sup> is a computer code for calculating the fractional release rate<sup>2</sup> of readily soluble radionuclides that are released from nuclear waste emplaced in water-saturated porous media. Waste placed in such environments will gradually dissolve. For many species such as actinides and rare earths, the process of dissolution is governed by the exterior flow field, and the chemical reaction rate or leaching rate.<sup>3</sup> However, for readily soluble species such as <sup>135</sup>Cs, <sup>137</sup>Cs and <sup>129</sup>I, it has been observed that their dissolution rates are rapid. UCB-NE-107 is a code for calculating the release rate at the waste/rock interface, to check compliance with the U. S. Nuclear Regulatory Commission's (USNRC) subsystem performance objective. It is an implementation of the analytic solution given below.

## THEORY

Small fractions of the more soluble species, e.g., cesium and iodine, are released from the UO<sub>2</sub> matrix during reactor operation and are present in UO<sub>2</sub> grain boundaries and pores and in fuel-cladding gaps. Upon contact with ground water that penetrates failed containers and fills voids in waste packages, these readily soluble species rapidly dissolve. Also there is continued slow dissolution of these same soluble species as the waste matrix dissolves in the void water. We assume that the surrounding porous rock is in direct contact with the well-mixed void liquid (Figure 1), that contains initially a specified quantity of these readily soluble species. The species migrate into the porous material under a concentration gradient. Assuming that advective transport in the near-field is small compared with diffusive transport, the space-time-dependent concentration  $N(x, t)$  in pore water is given by

$$K \frac{\partial N(x, t)}{\partial t} = D_f \frac{\partial^2 N(x, t)}{\partial x^2} - \lambda K N(x, t), \quad x > a, \quad t > 0 \quad (1)$$

where  $D_f$  is the species diffusion coefficient [L<sup>2</sup>/T] in the porous rock, and  $K$  is the species retardation coefficient.

The initial and boundary conditions are

$$N(x, 0) = 0, \quad x > a \quad (2)$$

$$N(\alpha, t) = N(t), \quad t \geq 0 \quad (3)$$

$$N(\infty, t) = 0, \quad t \geq 0 \quad (4)$$

where  $\alpha$  is the thickness of the "gap" filled with void water. Here  $N(t)$  is the time-dependent well-mixed concentration of the soluble species in the water in the void. To solve for  $N(t)$ , the mass balance in the void is

$$V \frac{dN(t)}{dt} = \dot{m}_f(t) - \dot{m}(t) - \lambda V N(t), \quad t > 0 \quad (5)$$

where  $\dot{m}_f(t)$  is the rate of dissolution of the species from the waste matrix into the void volume  $V$ , and  $\dot{m}(t)$  is the rate of species migration into the rock. To solve (5), we use the initial condition

$$N(0) = N^0 \quad (6)$$

where  $N^0$  is the initial concentration of the species in the void water.

Chambré<sup>4</sup> has obtained the solution to (1) through (6) resulting in the concentration of soluble species in the void as

$$N(t) = N^0 e^{-\lambda t} F(\beta^2 t) + \frac{1}{V} \int_0^t \dot{m}_f(t - \tau) e^{-\lambda \tau} F(\beta^2 \tau) d\tau, \quad t > 0 \quad (7)$$

where

$$F(\beta^2 t) \equiv e^{\beta^2} \operatorname{erfc} \sqrt{\beta^2 t}$$

and

$$\beta \equiv \sqrt{D_f K \epsilon^2 / \alpha^2}$$

The mass flux of the species into the rock is

$$\dot{m}(t) = -SD_f \epsilon \frac{\partial N(\alpha, t)}{\partial x}, \quad t > 0 \quad (8)$$

where  $S$  is the surface area of the interface between the void space and the rock ( $S \equiv V/\alpha$ ). Using (6) the solution to (7) is

$$\dot{m}(t) = N^0 \beta V e^{-\lambda t} \left\{ \frac{1}{\sqrt{\pi t}} - \beta F(\beta^2 t) \right\} + \beta \int_0^t \dot{m}_f(t - \tau) e^{-\lambda \tau} \left\{ \frac{1}{\sqrt{\pi \tau}} - \beta F(\beta^2 \tau) \right\} d\tau, \quad t > 0 \quad (9)$$

from which the fractional release rate can be calculated.

### ALGORITHM

UCB-NE-107 is an implementation of (9). Eq. (9) shows that the flux rate is the sum of the flux from the species released instantaneously in the gap, and the species released congruently from the waste matrix. UCB-NE-107 computes the two fluxes as well as the total flux, plus the fractional release rate, which is the total flux divided by the initial or 1000-year inventory. UCB-NE-107 also computes the concentration in the gap due to the initially released slug, and that from congruent release, as well as their sum.

The evaluation of

$$F(\beta^2 t) \equiv e^{\beta^2} \operatorname{erfc} \sqrt{\beta^2 t}$$

is by an adaptation of the rational approximation of the error function.<sup>5</sup>

### Numerical Illustration

Figure 2 shows the fractional release rates of  $^{135}\text{Cs}$ ,  $^{137}\text{Cs}$  and  $^{129}\text{I}$  based on (9). In this illustration we consider the release from a bare waste form exposed to ground water shortly after emplacement, conservatively neglecting the mass-transfer resistance due to containers and cladding. The lower curves show the solubility-limited dissolution rate from the waste matrix, assuming congruent dissolution and a uranium solubility of  $10^{-3}$  g/m<sup>3</sup>. A conservatively high diffusion coefficient of 0.12 m<sup>2</sup>/a is used throughout the analysis, and a porosity of 0.01 is assumed. The fractional release rates in Figure 2 are normalized to initial inventories. At  $t = 0$ , one percent of these species is assumed to dissolve into the water-filled void space of 0.45 m<sup>3</sup>. The calculated U. S. Nuclear Regulatory Commission fractional release rate limits on these species, based on initial inventories, are

$$^{135}\text{Cs} = 4.5 \times 10^{-5} \text{ per year}$$

$$^{137}\text{Cs} = 1.8 \times 10^{-10} \text{ per year}$$

$$^{129}\text{I} = 5.4 \times 10^{-4} \text{ per year}$$

With these parameters, the early contribution from gap inventory is more than  $10^7$ -fold greater than the contribution from the waste matrix. For early times, the fractional release rates of  $^{135}\text{Cs}$  and  $^{137}\text{Cs}$  are almost equal, but the  $^{137}\text{Cs}$  release rate decreases rapidly because of radioactive decay, whereas the release rate of  $3 \times 10^6$ -year  $^{135}\text{Cs}$  is characteristic of a stable species. Because of the assumed negligible sorption of iodine in the rock, the early fractional release rate of  $1.7 \times 10^7$ -year  $^{129}\text{I}$  is lower than the rates of strongly sorbing cesium, but the less rapid early depletion of iodine in the void water results in a greater fractional release rate of iodine after about 20 years.

The fractional release rate of  $^{137}\text{Cs}$  exceeds the USNRC release rate limit for several hundred years. The presence of metallic containers, and the time-distributed nature of container failure will likely help meet the requirement.

## INPUT

The input consists of 4 lines. UCB-NE-107 looks for a file called UCB107.dat as the input.

First line (free format)

RSP; equivalent waste-sphere radius (m)  
DF; diffusion coefficient ( $\text{m}^2/\text{a}$ )  
EPS; porosity of rock (-)  
VOL; volume of gap ( $\text{m}^3$ )  
WID; gap width (m)

Second line (free format)

AK; retardation coefficient of species (-)  
CONI; initial inventory of species in waste solid (g)  
SNGAP; initial concentration of species in gap ( $\text{g}/\text{m}^3$ )  
RAMBDA; decay constant of species ( $1/\text{a}$ )

Third line (free format)

ZK; retardation coefficient of the waste matrix (-)  
CONA; initial inventory of matrix in the waste (g)  
CS; solubility of dissolution controller ( $\text{g}/\text{m}^3$ )

Fourth line (free format)

NOPTION; option for output to FOR009  
TJUG; which initial inventory to use

An example input file follows.

```
.752, 3.1536d-4, .001, .45, .1,  
10., 1.38d3, 30.76, 2.31d-7,  
20., 5.d6, 1.d-3,  
2, 0.,
```

## OUTPUT

There are two output streams. The first is a tabular output, currently directed to the screen. The second is a stream of output data, for use by a X-Y plotter or plotting program such as TELL-A-GRAF.

The tabular output is currently directed to the screen by a

WRITE (\*,xxx)

statement. In a DEC/VAX computer this output can be directed to a file by a command such as

ASSIGN SYSS\$OUTPUT filename

Please note that the tabular output is 80 columns wide. The output first prints out most of the input data.

It also prints out the calculated leach time for the waste. Then in a table of 8 columns it lists

Time (a)

Concentration in the Gap Due to Instantaneous Release ( $\text{g}/\text{m}^3$ )  
Concentration in the Gap Due to Congruent Release from the Matrix ( $\text{g}/\text{m}^3$ )  
Total Concentration in the Gap ( $\text{g}/\text{m}^3$ )  
Mass Flux Rate at Waste/Rock Interface Due to Instantaneous Release ( $\text{g}/\text{a}$ )  
Mass Flux Rate at Waste/Rock Interface Due to Congruent Release ( $\text{g}/\text{a}$ )  
Total Mass Flux Rate at Waste/Rock Interface ( $\text{g}/\text{a}$ )  
Fractional Release Rate at Waste/Rock Interface ( $1/\text{a}$ )

The file FOR009.dat contains two columns of numbers. The first column is always the time in years. The second column depends on the value of NOPTION. If NOPTION=1, the second column contains the total fractional release rate (1/a). If NOPTION=2, the second column contains the fractional release rate (FRR) considering only the contribution of the instantaneously released species. If NOPTION=3, the second column contains the fractional release rate (FRR) considering the contribution from congruent dissolution from the waste matrix only. If NOPTION=4, the second column contains the total mass flux (g/a). If NOPTION=1, 2, 3 the fractional release rate has been divided by an inventory. If TJUG=0 : the fractional release rate is based on the initial inventory and if TJUG=1000, the fractional release rate is based on the thousand-year inventory.

### RESOURCE REQUIREMENTS

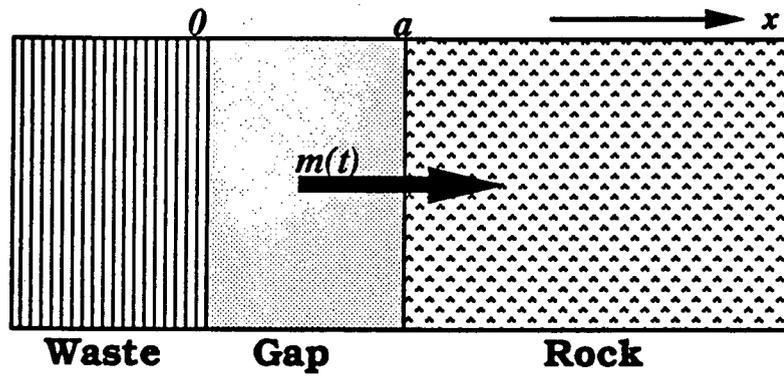
UCB-NE-107 is written in FORTRAN and does not call any outside libraries. It operates on DEC/VAX and DEC/UNIX machines.

### VERIFICATION & BENCHMARKING

UCB-NE-107 is an implementation of (9). It was verified through hand calculations. The correctness of UCB-NE-107 was also checked this way. Eq. (9) was given to several workers and they were asked to write computer programs independently. The outputs of the various programs are then compared or benchmarked. They gave identical results. The current UCB-NE-107 is in fact a combination of the best features of several programs.

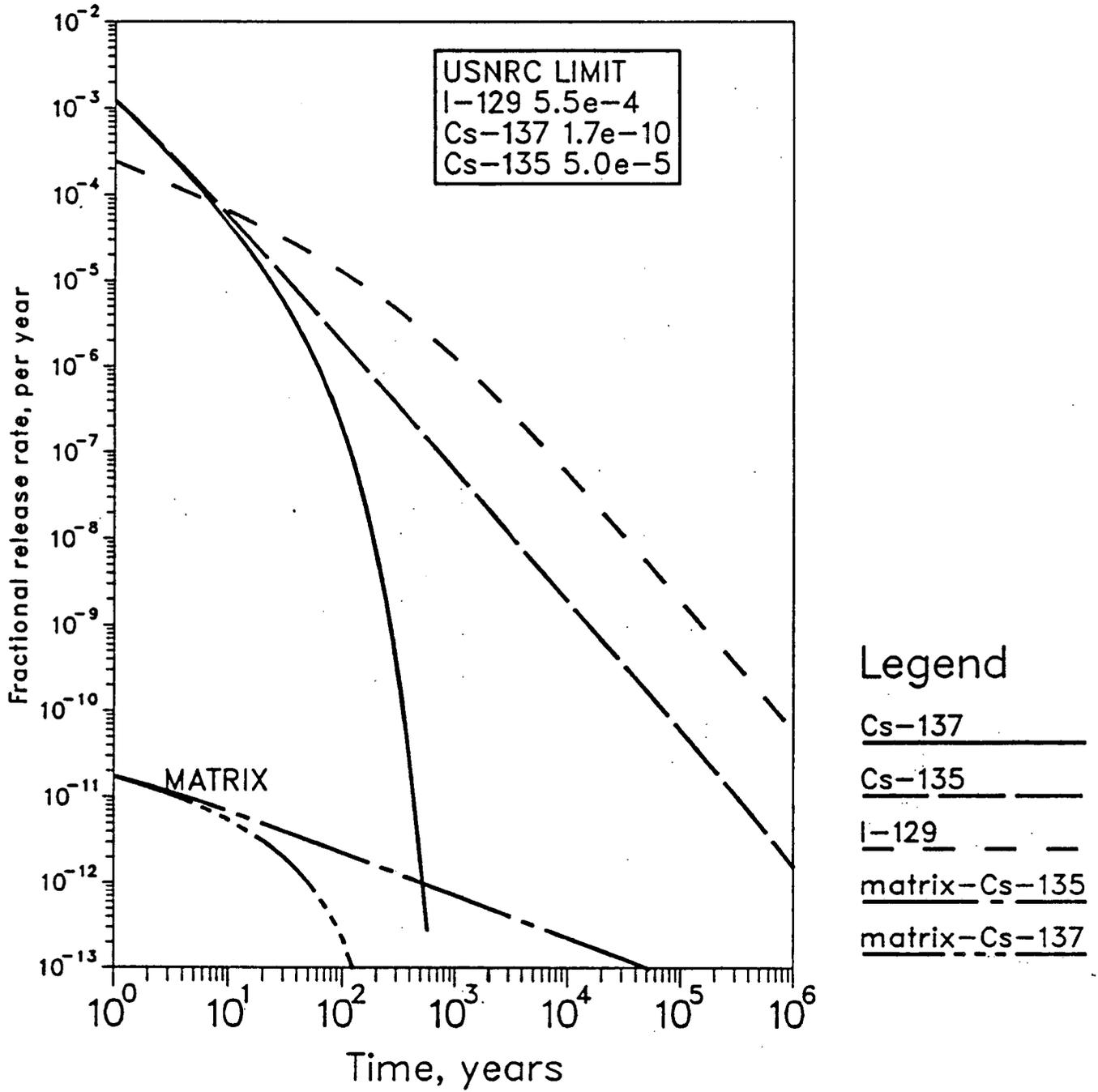
### REFERENCES

1. C. L. Kim, P. L. Chambré and T. H. Pigford, "Mass-Transfer Limited Release of a Soluble Waste Species," *Trans. Am. Nuc. Soc.*, 52, 80, 1986.
2. U. S. Nuclear Regulatory Commission, Disposal of High-Level Radioactive Wastes in Geologic Repositories, 10 *Code of Federal Regulations* 60.113(a)(1)(ii)(B).
3. P. L. Chambré, C. H. Kang, W. W.-L. Lee and T. H. Pigford, "The Role of Chemical Reaction in Waste-Form Performance," in M. J. Apted and R. E. Westerman (eds.), *Scientific Basis for Nuclear Waste Management XI*, 285, Pittsburgh, Materials Research Society, 1988
4. P. L. Chambré, to be published.
5. K. H. Haskell and R. E. Jones, 1978. *Brief Instructions for Using the Sandia Mathematical Subroutine Library (Version 7.2)*, SAND77-1441.



**Figure 1. Release of Soluble Species**

Figure 2. An Example of FOR009 Output Plotted Using TELL-A-GRAF.  
 Fractional Release Rates of Some Soluble Species.



APPENDIX A: CODE LISTING

```

c PROGRAM
c *****
c
c Gap Problem without Surface Reaction
c - in one dimensional geometry
c (x<0 - waste solid ; 0<x<a - gap ; x>a - rock)
c - gap is filled with water
c - concentration in the gap is uniform due to sufficient stirring
c - congruent dissolution from waste solid
c (solubility-limited diffusion-controlled dissolution)
c - diffusive mass transfer in the saturated porous rock
c
c >>Input<<
c First line (free format)
c RSP; equivalent waste-sphere radius (m)
c DF; diffusion coefficient (m**2/yr)
c EPS; porosity of rock (-)
c VOL; volume of gap (m**3)
c WID; gap width (m)
c Second line (free format)
c AK ; retardation coefficient of species (-)
c CONI; initial inventory of species in waste solid (g)
c SNGAP; initial concentration of species in gap (g/m**3)
c RAMBDA; decay constant of species (1/yr)
c Third line (free format)
c ZK ; retardation coefficient of dissolution controller (-)
c CONA ; initial inventory of controller in waste (g)
c CS ; solubility of dissolution controller (g/m**3)
c Fourth line (free format)
c if NOPTION=1 : write to FOR009 the total fractional release rate
c =2 : fractional release rate from gap only
c =3 : fractional release rate from solid only
c =4 : the total mass flux [g/yr]
c it TJUG=0 : based on initial inventory
c =1000 : thousand year inventory
c
c >>Output<<
c gap concentration of specie as a function of time
c mass flux from gap into surrounding porous rock
c
c Author --- Chang-Lak Kim
c Date --- Sep. 24, 1985
c Revision - Oct. 18, 1985
c Revised by Yongsoo Hwang July, 31 1987
c Second Revision - September, 23 1988, DERFL instead of DAWSON
c Third Revision - January 1989 (fsolid=0 after bleach)
c Fourth Revision - January 1989 by W. Lee, Output format
c
c Input File must be called UCB107.DAT
c 80-column output to screen unless SYS$OUTPUT assigned
c Output for TELL-A-GRAF are in FOR009 VIA THE OPTIONS
c FOR009 output in FRACTIONAL RELEASE RATES (FRR)
c *****
c IMPLICIT DOUBLE PRECISION (A-H,O-Z)
c EXTERNAL DAWSON,SDAWSON
c COMMON /UCB/ PI
c OPEN(5,FILE='UCB107.DAT',STATUS='UNKNOWN')
c READ(5,*) RSP,DF,EPS,VOL,WID
c READ(5,*) AK,CONI,SNGAP,RAMBDA

```

```

READ(5,*) ZK,CONA,CS
READ(5,*) NOPTION,TJUG

c
c
c   internal parameters
c
PI=DACOS(-1D0)
BETA=DSQRT(DF*AK*EPS*EPS/(WID*WID))
COEF=CS*CONI*DSQRT(ZK/AK)/CONA

c
c
c   matrix leach time calculation
c
c   revision 2
c   TLEACH is calculated based on the equation (2.32) in KIM's thesis
c   A1 is the initial controller mass
A1=CONA
A2=4.0d0*PI*EPS*RSP*DF*CS
AA=A1/A2
B1=8.0d0*EPS*CS*RSP*DSQRT(PI*DF*ZK)
B2=B1*B1-B1*DSQRT(B1*B1+4.0d0*A2*A1)
BB=B2/(2.0d0*A2*A2)
TLEACH=AA+BB

c
c   writing out input parameters
c
write(*,600) rsp,df,wid,vol,eps
600 format(24h INPUT DATA
*      24h Waste sphere radius      =, d9.3, 2h m/
*      24h diffusion coefficient    =, 1pd9.3,7h m**2/s/
*      24h gap width                =, d9.3,2h m/
*      24h volume of gap            =, d9.3,5h m**3/
*      24h porosity                 =, d9.3/)
write(*,601) ak,coni,sngap,rambda
601 format(52h retardation coefficient of the radionuclide      =,
*      1pd9.3/
*      52h initial inventory of the radionuclide in waste    =,
*      d9.3,2h g/
*      52h initial concentration of the radionuclide in gap  =,
*      d9.3,7h g/m**3/
*      52h decay constant of the radionuclide                 =,
*      d9.3,4h 1/s/)
write(*,602) zk,cona,cs,tleach
602 format(52h retardation coefficient of the controller      =,
*      1pd9.3/
*      52h initial inventory of the controller in waste      =,
*      d9.3,2h g/
*      52h solubility limit of the controller                 =,
*      d9.3,7h g/m**3/
*      52h matrix leach time                                   =,
*      d9.3, 2h s/)
WRITE (*,606)
606 FORMAT(' TIME CONCENTRATION FROM ',
*      FLUX FROM')
WRITE (*,603)
603 format(' years gap matrix TOTAL gap ',
*      27h matrix TOTAL FRR/1x,80(1h-))
c

```

```

c   beginning of calculations
c
  IF(TLEACH.GE.1.0d7) GO TO 231
  TDEAD=TLEACH
  go to 232
231 TDEAD=1.0d7
232 CONTINUE
  ALL=1.d0
  SUP=-1.d0
110 TSE=ALL*10.0d0**SUP
c
  RT=DEXP(-RAMBDA*TSE)
  BT=BETA*BETA*TSE
  CGAP=SNGAP*DAWSON(BT)*RT
  CALL SDAWSON(BETA,TSE,ANSWER)
  FGAP=BETA*VOL*SNGAP*RT*ANSWER
  CSOLID=COEF*(1.-DAWSON(BT))*RT
  IF(TSE.GT.TLEACH) GO TO 45671
  FSOLID=BETA*BETA*VOL*COEF*RT*DAWSON(BT)
  GO TO 45672
45671 FSOLID=0.0d0
45672 CONTINUE
  CN=CGAP+CSOLID
  IF(NOPTION.EQ.4) GO TO 331
  IF(NOPTION.EQ.1) GO TO 331
  IF(NOPTION.EQ.2) GO TO 332
  FLUX=FSOLID
  GO TO 333
332 FLUX=FGAP
  GO TO 333
331 FLUX=FSOLID+FGAP
  IF(NOPTION.EQ.4) GO TO 934
  GO TO 333
934 WRITE(9,*) TSE,FLUX
  GO TO 590
333 CONTINUE
  SONE=RAMBDA*TJUG
  FINI=DEXP(-SONE)*CONI
  FRAC=FLUX/FINI
  WRITE(9,*) TSE,FRAC
  WRITE(*, 004) tSE, cgap, csolid, Cn, fgap, fsolid, fgap+fsolid,
  *(fgap+fsolid)/fini
590 CONTINUE
  IF(TSE.GE.TDEAD) GO TO 120
  IF(ALL.GE.9.0d0) GO TO 130
  ALL=ALL+1.0d0
004 format(1pd10.3,7d10.2)
  GO TO 110
130 ALL=1.0d0
  SUP=SUP+1.0d0
  GO TO 110
120 STOP
  END

```

```

c
c
c
c
c

```

```

REAL*8 FUNCTION DAWSON(X)
IMPLICIT DOUBLE PRECISION (A-H,O-Z)

```

```

EXTERNAL ERFL
XXZ=DSQRT(X)
DAWSON=ERFL(XXZ)
RETURN
END

```

```

c
SUBROUTINE SDAWSON(BETA,TSE,ANSWER)
IMPLICIT DOUBLE PRECISION (A-H,O-Z)
EXTERNAL ERFL
COMMON /UCB/ PI
CCX=BETA*BETA*TSE
CCX=DSQRT(CCX)
ANSWER=1.0d0/DSQRT(PI*TSE)-BETA*ERFL(CCX)
RETURN
END

```

```

c
FUNCTION ERFL(XS)
IMPLICIT DOUBLE PRECISION (A-H,O-Z)
REAL*8 P(6),Q(6),R(4),S(4)
DATA P/22.898992851659D0,26.094746956075D0,14.571898596926D0,
*4.2677201070898D0,.56437160686381D0,-6.0858151959688D-6/,
*Q /22.898985749891D0,51.933570687552D0,50.273202863803D0,
*26.288795758761D0,7.5688482293618D0,1D0/,
*R /-1.21308276389978D-2,-.119903955268146D0,-.243911029488626D0,
*-3.24319519277746D-2/,
*S /4.3002664345277D-2,.489552441961437D0,1.43771227937118D0,1D0/
DATA SQPI/.564189583547756D0/
AX=DABS(XS)
IF(AX.GT.4.0) GO TO 10
TMP=P(1)+AX*(P(2)+AX*(P(3)+AX*(P(4)+AX*(P(5)+AX*(P(6))))))
A=TMP/(Q(1)+AX*(Q(2)+AX*(Q(3)+AX*(Q(4)+AX*(Q(5)+AX*(Q(6)))))
ERFL=DSIGN(A,XS)
GO TO 11
10 CONTINUE
X2=1./AX/AX
B=X2*(R(1)+X2*(R(2)+X2*(R(3)+X2*(R(4)))))/(S(1)+X2*(S(2)+X2*
*(S(3)+X2*(S(4))))
A=(SQPI+B)/AX
ERFL=DSIGN(A,XS)
11 CONTINUE
RETURN
END

```

## APPENDIX B: SAMPLE OUTPUT

\*Output from GAP

Jan. 16, 1989

## INPUT DATA

Waste sphere radius =0.752D+00 m  
 diffusion coefficient =3.154D-04 m\*\*2/a  
 gap width =1.000D-01 m  
 volume of gap =4.500D-01 m\*\*3  
 porosity =1.000D-03  
  
 retardation coefficient of the radionuclide =1.000D+01  
 initial inventory of the radionuclide in waste =1.380D+03 g  
 initial concentration of the radionuclide in gap =3.076D+01 g/m\*\*3  
 decay constant of the radionuclide =2.310D-07 1/a  
  
 retardation coefficient of the controller =2.000D+01  
 initial inventory of the controller in waste =5.000D+06 g  
 solubility limit of the controller =1.000D-03 g/m\*\*3  
 matrix leach time =1.678D+15 a

TIME years	CONCENTRATION FROM			FLUX FROM			FRR
	gap	matrix	TOTAL	gap	matrix	TOTAL	
1.000D-01	3.08D+01	7.81D-11	3.08D+01	1.39D-02	5.54D-14	1.39D-02	1.00D-05
2.000D-01	3.08D+01	1.10D-10	3.08D+01	9.80D-03	5.54D-14	9.80D-03	7.10D-06
3.000D-01	3.07D+01	1.35D-10	3.07D+01	8.00D-03	5.54D-14	8.00D-03	5.80D-06
4.000D-01	3.07D+01	1.56D-10	3.07D+01	6.93D-03	5.54D-14	6.93D-03	5.02D-06
5.000D-01	3.07D+01	1.75D-10	3.07D+01	6.20D-03	5.54D-14	6.20D-03	4.49D-06
6.000D-01	3.07D+01	1.91D-10	3.07D+01	5.66D-03	5.54D-14	5.66D-03	4.10D-06
7.000D-01	3.07D+01	2.07D-10	3.07D+01	5.24D-03	5.54D-14	5.24D-03	3.80D-06
8.000D-01	3.07D+01	2.21D-10	3.07D+01	4.90D-03	5.54D-14	4.90D-03	3.55D-06
9.000D-01	3.07D+01	2.34D-10	3.07D+01	4.62D-03	5.54D-14	4.62D-03	3.35D-06
1.000D+00	3.07D+01	2.47D-10	3.07D+01	4.38D-03	5.54D-14	4.38D-03	3.17D-06
2.000D+00	3.07D+01	3.49D-10	3.07D+01	3.10D-03	5.53D-14	3.10D-03	2.24D-06
3.000D+00	3.07D+01	4.28D-10	3.07D+01	2.53D-03	5.53D-14	2.53D-03	1.83D-06
4.000D+00	3.07D+01	4.94D-10	3.07D+01	2.19D-03	5.53D-14	2.19D-03	1.59D-06
5.000D+00	3.07D+01	5.52D-10	3.07D+01	1.96D-03	5.53D-14	1.96D-03	1.42D-06
6.000D+00	3.07D+01	6.05D-10	3.07D+01	1.79D-03	5.53D-14	1.79D-03	1.29D-06
7.000D+00	3.07D+01	6.53D-10	3.07D+01	1.65D-03	5.53D-14	1.65D-03	1.20D-06
8.000D+00	3.07D+01	6.98D-10	3.07D+01	1.55D-03	5.53D-14	1.55D-03	1.12D-06
9.000D+00	3.07D+01	7.41D-10	3.07D+01	1.46D-03	5.53D-14	1.46D-03	1.06D-06
1.000D+01	3.07D+01	7.81D-10	3.07D+01	1.38D-03	5.53D-14	1.38D-03	1.00D-06
2.000D+01	3.07D+01	1.10D-09	3.07D+01	9.76D-04	5.52D-14	9.76D-04	7.07D-07
3.000D+01	3.07D+01	1.35D-09	3.07D+01	7.96D-04	5.52D-14	7.96D-04	5.77D-07
4.000D+01	3.06D+01	1.56D-09	3.06D+01	6.89D-04	5.52D-14	6.89D-04	4.99D-07
5.000D+01	3.06D+01	1.74D-09	3.06D+01	6.16D-04	5.51D-14	6.16D-04	4.46D-07
6.000D+01	3.06D+01	1.91D-09	3.06D+01	5.62D-04	5.51D-14	5.62D-04	4.07D-07
7.000D+01	3.06D+01	2.06D-09	3.06D+01	5.20D-04	5.51D-14	5.20D-04	3.77D-07
8.000D+01	3.06D+01	2.20D-09	3.06D+01	4.86D-04	5.51D-14	4.86D-04	3.52D-07
9.000D+01	3.06D+01	2.34D-09	3.06D+01	4.58D-04	5.51D-14	4.58D-04	3.32D-07
1.000D+02	3.06D+01	2.46D-09	3.06D+01	4.34D-04	5.50D-14	4.34D-04	3.15D-07
2.000D+02	3.05D+01	3.47D-09	3.05D+01	3.06D-04	5.49D-14	3.06D-04	2.22D-07
3.000D+02	3.04D+01	4.25D-09	3.04D+01	2.49D-04	5.48D-14	2.49D-04	1.80D-07
4.000D+02	3.04D+01	4.90D-09	3.04D+01	2.15D-04	5.47D-14	2.15D-04	1.56D-07
5.000D+02	3.03D+01	5.47D-09	3.03D+01	1.92D-04	5.46D-14	1.92D-04	1.39D-07
6.000D+02	3.03D+01	5.98D-09	3.03D+01	1.75D-04	5.45D-14	1.75D-04	1.27D-07
7.000D+02	3.02D+01	6.46D-09	3.02D+01	1.61D-04	5.45D-14	1.61D-04	1.17D-07
8.000D+02	3.02D+01	6.90D-09	3.02D+01	1.51D-04	5.44D-14	1.51D-04	1.09D-07
9.000D+02	3.02D+01	7.31D-09	3.02D+01	1.42D-04	5.43D-14	1.42D-04	1.03D-07
1.000D+03	3.01D+01	7.70D-09	3.01D+01	1.34D-04	5.43D-14	1.34D-04	9.74D-08
2.000D+03	2.99D+01	1.08D-08	2.99D+01	9.38D-05	5.38D-14	9.38D-05	6.80D-08

3.000D+03	2.97D+01	1.32D-08	2.97D+01	7.58D-05	5.35D-14	7.58D-05	5.49D-08
4.000D+03	2.95D+01	1.51D-08	2.95D+01	6.51D-05	5.32D-14	6.51D-05	4.72D-08
5.000D+03	2.94D+01	1.69D-08	2.94D+01	5.78D-05	5.29D-14	5.78D-05	4.19D-08
6.000D+03	2.93D+01	1.84D-08	2.93D+01	5.24D-05	5.27D-14	5.24D-05	3.80D-08
7.000D+03	2.91D+01	1.98D-08	2.91D+01	4.82D-05	5.25D-14	4.82D-05	3.49D-08
8.000D+03	2.90D+01	2.11D-08	2.90D+01	4.48D-05	5.23D-14	4.48D-05	3.25D-08
9.000D+03	2.89D+01	2.24D-08	2.89D+01	4.20D-05	5.21D-14	4.20D-05	3.05D-08
1.000D+04	2.88D+01	2.35D-08	2.88D+01	3.97D-05	5.19D-14	3.97D-05	2.87D-08
2.000D+04	2.81D+01	3.25D-08	2.81D+01	2.69D-05	5.05D-14	2.69D-05	1.95D-08
3.000D+04	2.75D+01	3.91D-08	2.75D+01	2.12D-05	4.95D-14	2.12D-05	1.54D-08
4.000D+04	2.70D+01	4.45D-08	2.70D+01	1.79D-05	4.86D-14	1.79D-05	1.30D-08
5.000D+04	2.65D+01	4.91D-08	2.65D+01	1.56D-05	4.78D-14	1.56D-05	1.13D-08
6.000D+04	2.61D+01	5.32D-08	2.61D+01	1.39D-05	4.71D-14	1.39D-05	1.01D-08
7.000D+04	2.58D+01	5.68D-08	2.58D+01	1.26D-05	4.64D-14	1.26D-05	9.17D-09
8.000D+04	2.55D+01	6.01D-08	2.55D+01	1.16D-05	4.59D-14	1.16D-05	8.41D-09
9.000D+04	2.52D+01	6.31D-08	2.52D+01	1.07D-05	4.53D-14	1.07D-05	7.79D-09
1.000D+05	2.49D+01	6.58D-08	2.49D+01	1.00D-05	4.48D-14	1.00D-05	7.26D-09
2.000D+05	2.26D+01	8.59D-08	2.26D+01	6.16D-06	4.07D-14	6.16D-06	4.46D-09
3.000D+05	2.09D+01	9.85D-08	2.09D+01	4.50D-06	3.77D-14	4.50D-06	3.26D-09
4.000D+05	1.96D+01	1.07D-07	1.96D+01	3.54D-06	3.53D-14	3.54D-06	2.57D-09
5.000D+05	1.84D+01	1.14D-07	1.84D+01	2.91D-06	3.32D-14	2.91D-06	2.11D-09
6.000D+05	1.74D+01	1.19D-07	1.74D+01	2.46D-06	3.14D-14	2.46D-06	1.78D-09
7.000D+05	1.65D+01	1.22D-07	1.65D+01	2.11D-06	2.98D-14	2.11D-06	1.53D-09
8.000D+05	1.57D+01	1.25D-07	1.57D+01	1.85D-06	2.83D-14	1.85D-06	1.34D-09
9.000D+05	1.50D+01	1.27D-07	1.50D+01	1.63D-06	2.70D-14	1.63D-06	1.18D-09
1.000D+06	1.43D+01	1.28D-07	1.43D+01	1.45D-06	2.57D-14	1.45D-06	1.05D-09
2.000D+06	9.52D+00	1.25D-07	9.52D+00	6.03D-07	1.71D-14	6.03D-07	4.37D-10
3.000D+06	6.69D+00	1.10D-07	6.69D+00	3.16D-07	1.21D-14	3.16D-07	2.29D-10
4.000D+06	4.84D+00	9.36D-08	4.84D+00	1.84D-07	8.71D-15	1.84D-07	1.33D-10
5.000D+06	3.55D+00	7.79D-08	3.55D+00	1.14D-07	6.40D-15	1.14D-07	8.24D-11
6.000D+06	2.64D+00	6.41D-08	2.64D+00	7.31D-08	4.75D-15	7.31D-08	5.30D-11
7.000D+06	1.98D+00	5.24D-08	1.98D+00	4.83D-08	3.56D-15	4.83D-08	3.50D-11
8.000D+06	1.49D+00	4.26D-08	1.49D+00	3.27D-08	2.69D-15	3.27D-08	2.37D-11
9.000D+06	1.13D+00	3.45D-08	1.13D+00	2.24D-08	2.03D-15	2.24D-08	1.63D-11
1.000D+07	8.60D-01	2.78D-08	8.60D-01	1.56D-08	1.55D-15	1.56D-08	1.13D-11

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